



Waste and Cleanup Risk Assessment

Dose Compliance Concentrations for Radionuclides in Buildings (BDCC)

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BDCC User's Guide

[PDF of User's Guide.](#)

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Welcome to the EPA's Superfund Dose Compliance Concentrations for Radionuclides in Buildings (BDCC) user's guide. Here you will find descriptions, equations and default exposure parameters used to calculate the dose-based BDCCs. Additional guidance is also provided on sources of parameters and proper BDCC use. It is suggested that users read the [BDCC FAQ](#) page before proceeding. The user guide is extensive so please use the "Open All Sections" and "Close All Sections" links below as needed. Individual sections can be opened and closed by clicking on the section titles. Before proceeding through the user's guide please read the Disclaimer.

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Disclaimer

This guidance document sets forth recommended approaches based on EPA's best thinking to date with respect to dose assessment for response actions at CERCLA sites. This document does not establish binding rules. Alternative approaches for dose assessment may be found to be more appropriate at specific sites (e.g., where site circumstances do not match the underlying assumptions, conditions and models of the guidance). The decision whether to use an alternative approach and a description of any such approach should be placed in the Administrative Record for the site. Accordingly, comments are received at individual sites questioning the use of the approaches recommended in this guidance, the comments should be considered and an explanation provided for the selected approach.

The policies set out in the Radionuclide ARAR BDCC User Guide provide guidance to EPA staff. It also provides guidance to the public and regulated community on how EPA intends the National Oil and Hazardous Substances Pollution Contingency Plan (NCP) be implemented. EPA may change this guidance in the future, as appropriate. This calculator is intended for use by risk assessors, health physicists and other qualified environmental protection specialists.

It should also be noted that calculating a BDCC addresses neither human cancer risk or noncancer toxicity, nor potential ecological risk. Of the radionuclides generally found, at CERCLA sites, only uranium has potentially significant noncancer toxicity. When assessing sites with uranium as a contaminant, it may also be necessary to consider the noncancer toxicity of uranium, using other tools, such as EPA's Regional Screening Levels (RSLs) for Chemical Contaminants at Superfund Sites electronic calculator for uranium in soil, water, or air, and the [WTC](#) for uranium inside buildings. EPA's [DCC](#) Calculator should be used to assess radionuclide dose in soil, water, and air, and the [SDCC](#) Calculator for radionuclide dose for hard outside surfaces. EPA's [PRG](#) Calculator should be used to assess radionuclide cancer risk for soil, water, and air, [BPRG](#) Calculator for radionuclide cancer risk inside buildings, and the [SPRG](#) Calculator for radionuclide cancer for hard outside surfaces. Similarly, some sites with radiological contaminants in sensitive ecological settings may also need to be evaluated for potential ecological risk. EPA's guidance "[Ecological Risk Assessment Guidance for Superfund: Process for Designing and Conducting Ecological Risk Assessment](#)" contains an eight step process for using benchmarks for ecological effects in the remedy selection process.

1. Introduction

Generally, these recommended Dose Compliance Concentrations for Radionuclides inside Buildings (BDCCs) are reasonable maximum exposure (RME) risk concentrations derived from standardized equations that combine exposure information and toxicity information in the form of dose conversion factors (DCFo). Recommended BDCCs are presented for residential and indoor worker exposure.

The BDCC is a tool that the U.S. Environmental Protection Agency developed to help standardize the evaluation and cleanup of radioactively contaminated sites at which doses are being assessed. This guidance provides a methodology for radiation professionals to calculate dose-based, site-specific, dose compliance concentrations (BDCCs) for radionuclides inside of buildings while complying with a dose-based standard as an ARAR. This guidance supersedes the dose assessment methodology contained in the "Risk Assessment Guidance for Superfund Volume I Human Health Evaluation Manual (Part A) (EPA/540/1-89/002).

A number of different radiation standards may be used as Applicable or Relevant and Appropriate Requirements (ARARs) to establish cleanup levels at a site. Cleanup levels may be based on a number of Federal or State ARARs. Federal standards expressed in terms of dose that are potential ARARs at CERCLA sites include 40 CFR Part 190, "Environmental Radiation Protection Standards for Nuclear Power Operations," 40 CFR Part 191, "Environmental Radiation Protection Standards for Management and Disposal of Spent Nuclear Fuel, High-Level and Transuranic Radioactive Wastes," or 10 CFR Part 61, "Licensing Requirements for Land Disposal of Radioactive Waste," among others.

One set of radiation standards consists of a combination of whole body and critical organ dose annual limits, generally either (1) 25 mrem to the whole body, 75 mrem to the thyroid, and 25 mrem to any other critical organ besides the thyroid, or (2) 25 mrem/yr to the whole body and 75 mrem/yr to any critical organ

(including the thyroid). A third set of standards consists of a single limit (e.g., 10 mrem/yr). The type of dose limit used in the standard would be the same type of dose methodology used for purposes conducting dose assessment to demonstrate ARAR compliance.

The changes in the dose limits reflect an evolution over time in the approach to dose limitation and in the methods used to calculate doses. The first two radiation protection standards listed above—the 25/75/25 and 25/75 mrem annual dose limits—are based on the older, critical organ concept of dose limitation. This approach limits dose and long-term effects to a specific target tissue or organ (e.g., the thyroid), the most radiosensitive tissue or organ, or the tissue or organ receiving the highest dose. Under this approach, introduced in 1959 by the International Commission on Radiological Protection (ICRP) in its Publication 2, "Report of Committee II on Permissible Dose for Internal Radiation," (ICRP, 1959) the dose to an organ from internally-deposited radionuclides is calculated separately from the dose due to external exposure, and the whole body is essentially treated as one of the critical organs.

Standards were then based on the effective dose equivalent concept of dose limitation introduced in 1977 by the ICRP in its Publication 26, "Recommendations of the International Commission on Radiological Protection" (ICRP, 1977). The effective dose equivalent approach accounts for the differences in the cancer induction rates in organs and tissues subjected to equal doses of radiation and normalizes these doses and effects on a whole body basis. Under this approach, the effective dose equivalent dose is calculated as the weighted sum of the committed dose equivalents (from ingested and inhaled radionuclides) and the dose equivalent (for external exposure from photon-emitting radionuclides) to all organs and tissues. The weighting factors used in these calculations are organ-specific and correspond to the fractional contribution of each organ or tissue to the total risk of fatal cancers when the body is uniformly irradiated. Thus, the summation of all organ and tissue factors is equal to one.

ICRP has since updated the effective dose equivalent concept with the introduction of effective dose quantity in its Publication 60, "1990 recommendations of the International Commission on Radiological Protection" (ICRP, 1991). The effective dose quantity is similar to the effective dose equivalent approach while incorporating updated scientific information in the dose conversion factors. Effective dose quantity incorporates a greater number of organs and updated information on organ-specific risk, and age-specific dose coefficients for internal exposure which incorporate new physiologically-based biokinetic models.

ICRP Publication 107 (ICRP 2008) provides an electronic database of the physical data needed in calculations of radionuclide-specific protection and operational quantities. This database supersedes the data of ICRP 38 and will be used in future ICRP publications of dose coefficients for the intake of or exposure to radionuclides in the workplace and the environment.

The purpose of this document is provide guidance to EPA personnel for the calculation of release criteria based on regulations promulgated under various methods of dose calculation. This guidance will relate these dose limits to a single measure, cleanup concentration. This guidance will assist RPMs in making decisions at these sites.

Users of this guidance should note that the use of this calculator to develop dose compliance concentrations for some dose-based ARARs does not affect the CERCLA requirement to comply with all other Federal and State ARARs at a site (e.g., 40 CFR 141.66, 40 CFR 192.12). ARARs are determined site-specifically. For a list of "Likely Federal Radiation Applicable or Relevant and Appropriate (ARARs)", see Attachment A of EPA's guidance "[Establishment of Cleanup Levels for CERCLA sites with Radioactive Contamination.](#)" For additional guidance documents on compliance with ARARs at radioactively contaminated sites, go to the following webpage:

<http://www.epa.gov/superfund/health/contaminants/radiation/radarars.htm>

This website uses DCFo provided by the [Center for Radiation Protection Knowledge](#). The main report is [Calculations of Slope Factors and Dose Coefficients](#) and the tables of DCFo are in a separate [appendix](#). The DCFo are combined with "standard" exposure factors to estimate contaminant concentrations inside buildings that attain compliance with a dose-based ARAR. Exceeding a BDCC usually suggests that further evaluation of the potential dose is appropriate. The BDCC concentrations presented on this website can be used to screen pollutants in environmental media, trigger further investigation, and provide initial cleanup goals, if applicable. BDCCs should be applied in accordance with guidance from EPA Regions.

2. Understanding the BDCC Website

2.1 General Considerations

BDCCs are isotope activities that correspond to fixed levels of dose (e.g., mrem) inside a building. Dose conversion factors (DCFo), or "dose coefficients", for a given radionuclide represent the dose equivalent per unit intake (i.e., ingestion or inhalation) or external exposure of that radionuclide. These DCFo are used to convert a radionuclide concentration in soil, air, water, or foodstuffs to a radiation dose. DCFo may be specified for specific body organs or tissues of interest, or as a weighted sum of individual organ dose, termed the effective dose equivalent. These DCFo may be multiplied by the total activity of each radionuclide inhaled or ingested per year, or the external exposure concentration to which a receptor may be exposed, to estimate the dose equivalent to the receptor.

Inhalation risk coefficients are tabulated separately for each of the three lung absorption types considered in the lung model currently recommended by the International Commission on Radiological Protection (ICRP), and, where appropriate, for inhalation of radionuclides in vapor or gaseous forms.

The designations "F", "M", and "S" presented in the Radionuclide Table under the heading "ICRP Lung Type" refer to the lung absorption type for inhaled particulate radionuclides, expressed as fast (F), medium (M), or slow (S), as used in the current ICRP model of the respiratory tract. The inhalation dose conversion factor value tabulated in the Radionuclide Table for each radionuclide have been selected based on the following guidelines: (1) For those elements where Table 4.1 of Federal Guidance Report No. 13 (and Table 2 of ICRP Publication 72) specifies a recommended default lung absorption type for particulates, the inhalation dose conversion factor for that type is tabulated in the Radionuclide Table for each radioisotope of that element. (2) For those elements where no specific lung absorption type is recommended and multiple types are indicated as plausible choices, the inhalation dose conversion factor reported in the Radionuclide Table for each radioisotope of that element is the maximum of the values for each of the plausible lung absorption types. (3) Where Federal Guidance Report No. 13 specifies risk coefficients for multiple chemical forms of certain elements (tritium, carbon, sulfur, iodine, and mercury), the inhalation dose conversion factor value for the form estimated to pose the maximum risk is reported in the Radionuclide Table in most cases.

Inhaled particulates are assumed to have an activity median aerodynamic diameter (AMAD) of 1 μm , as recommended by the ICRP for consideration of environmental exposures in the absence specific information physical characteristics of the aerosol. Where appropriate, radionuclides may be present in gas or vapor form, are designated by "G" and "V", respectively; such radionuclides include tritium, carbon, sulfur, nickel, ruthenium, iodine, tellurium, and mercury.

The most likely exposure scenarios and exposure assumptions are included in the equations on this website: Resident and Indoor Worker.

The recommended BDCCs are generated with [standard exposure route equations](#) using EPA DCFo and parameters.

2.2 Dose Conversion Factors (DCFo)

Users of this calculator tool should choose the DCFo (International Commission on Radiological Protection (ICRP) 30, 60 or 107) required by the ARAR. If DCFo are not specified within the regulation (for example, the Code of Federal Regulations for a federal standard that is being complied with as an ARAR), then users should generally choose ICRP 107 DCFo. This recommendation is consistent with the guidance contained in "Use of IRIS Values in Superfund Risk Assessment" (OSWER 9285.7-16) for EPA to evaluate dose based upon its best scientific judgment. For further discussion of the scientific differences between ICRP 30 and 60 methodologies, see "Dosimetric Significance of the ICRP's Updated Guidance and Models, 1989-2003, and Implications for U.S. Federal Guidance" (August 2003, ORNL/TM-2003/207). <http://ordose.ornl.gov/documents/ornltm2003-207.pdf>. For a discussion of the impacts of the ICRP 107 nuclear decay data, see [Impact of the New Nuclear Decay Data of ICRP Publication 107 on Inhalation Dose Coefficients for Workers](#).

EPA classifies all radionuclides as Group A carcinogens ("carcinogenic to humans"). Group A classification is used only when there is sufficient evidence from epidemiologic studies to support a causal association between exposure to the agents and cancer. The [appendix radionuclide table](#), from the [Center for Radiation Protection Knowledge](#), lists ingestion, inhalation and external exposure dose coefficients for radionuclides in conventional units of picocuries (pCi). Ingestion and inhalation dose coefficients are central estimates in a linear model of the age-averaged, lifetime attributable radiation cancer incidence (fatal and nonfatal cancer) dose per unit of activity inhaled or ingested, expressed as mrem/pCi. External exposure dose coefficients are central estimates of lifetime attributable radiation dose for each year of exposure to external radiation from photon-emitting radionuclides distributed uniformly in a thick layer of soil, and are expressed as *mrem/year per pCi/gram soil*. External exposure dose coefficients can also be used which have units of *mrem/year per pCi/cm² soil*. When combined with site-specific media concentration data and appropriate exposure assumptions, dose coefficients can be used to estimate annual dose to members of the general population due to radionuclide exposures. EPA currently provides guidance on inhalation risk assessment in [RAGS Part F](#) (Risk Assessment Guidance for Superfund Volume I: Human Health Evaluation Manual (Part F, Supplemental Guidance for Inhalation Risk Assessment)). This guidance only addresses chemicals. The development of inhalation dose coefficients for radionuclides differs from the guidance presented in RAGS Part F for development of inhalation unit risk (IUR) values for chemicals.

The DCFo from the [Center for Radiation Protection Knowledge](#) differ from the values presented in [FGR 12 CD supplement](#). The DCFo were calculated using ORNL's DCAL software in the manner of Federal Guidance Report 12 and 13. The radionuclides presented are those provided in the International Commission on Radiological Protection (ICRP) [Publication 107](#). This document contains a revised database of nuclear decay data (energies and intensities of emitted radiations, physical half-lives and decay modes) for 1,252 naturally occurring and man-made radionuclides. ICRP Publication 107 supersedes the previous database, ICRP Publication 38 published in 1983. In addition to radionuclides in ICRP Publication 107, the assembled data files or tables contain entries that include the contribution of daughter products in secular equilibrium with their longer-lived parents. These entries are identified by the addition of "+D" or "+E" to the nuclide name where "+D" refers to a 100 year period of progeny ingrowth and "+E" is 1000 years.

2.2.1 ICRP 30

Unlike ICRP 2 which did not calculate DCFo per se, ICRP 30 does present DCFo which may be used to calculate either organ dose equivalent or effective dose equivalent for ingestion and inhalation. For each radionuclide, ICRP 30 provides values for the organ dose equivalent conversion factors, $hT,50$, and the effective dose equivalent conversion factor, $hE,50$ (calculated using the organ weighting factors w_T). These values are also presented in [Federal Guidance Report No. 11](#). Organ DCFo are provided for those organs which have specific weighting factors, namely the gonads, breast, red marrow, lungs, thyroid, and bone surfaces. Organ DCFo are also given for the remainder, which includes the five remaining tissues which receive the next highest doses. These include the liver, kidneys, spleen, brain, small intestine, upper large intestine, lower large intestine, etc.

Organ dose equivalent conversion factors and effective dose equivalent conversion factors for all radionuclides selected for this analysis are provided in Attachment A, Table A.2 (inhalation) and Table A.3 (ingestion). These values, in units of mrem/pCi, have been taken from Tables 2.1 and 2.2 respectively of [Federal Guidance Report No. 11](#).

2.2.2 ICRP 60

ICRP 60 also presents DCFo. This is the document on which most of the world's radiation standards are based. ICRP 60 is similar to ICRP 30 except that it is based on more recent findings. Basically, there were more cancers observed in the Japanese populations exposed to radiation in the bombings, and so risk estimates increased. There were also reevaluations of the radiation dose calculations. These values are also presented in [Federal Guidance Report No. 13](#).

The spontaneous fission isotopes are not in FGR-13. They are released in ICRP 72. ICRP 72 is analogous to the FGR 13 CD that contains most of the same values. Cf-252, Cf-254, Cm-248, Cm-250 and Pu-244 are the isotopes that decay by spontaneous fission at greater than 0.1%.

The use of dose conversion factors of ICRP 60/72 is mandated in the European Union by [European Council Directive 96/29](#) of May 13, 1996. If requested, NRC can grant a licensee an exemption to use the new dosimetric data of the ICRP; e.g. ICRP 68 for occupational exposures. In accordance with a June 8, 2007 [Federal Register](#) notice, DOE no longer requires a facility to get an exception and they are able to use ICRP 68 dosimetric data for occupational exposure. Non-regulatory studies (e.g., risk assessments) use the technically best available dose coefficients which are those of the recent ICRP Publications. In addition, the IAEA in its Safety Series has adopted the ICRP Publication 60 Recommendations and the subsequent dose coefficients. For example, the dose coefficients of ICRP Publication 68 are contained in the IAEA Safety Guide entitled "Assessment of Occupational Exposure Due to Intakes of Radionuclides" RS-G-1.2 issued in 1999.

2.2.3 ICRP 107

ICRP Publication 107 (ICRP 2008) provides an electronic database of the physical data needed in calculations of radionuclide-specific protection and operational quantities. This database supersedes the data of ICRP 38 and will be used in future ICRP publications of dose coefficients for the intake of or exposure to radionuclides in the workplace and the environment.

The database contains information on the half-lives, decay chains, and yields and energies of radiations emitted in nuclear transformations of 1252

radionuclides of 97 elements. The CD accompanying the publication provides electronic access to complete tables of the emitted radiations, as well as the beta and neutron spectra. The database has been constructed such that user-developed software can extract the data needed for further calculations of a radionuclide of interest. A Windows-based application is provided to display summary information on a user-specified radionuclide, as well as the general characterization of the nuclides contained in the database. In addition, the application provides a means by which the user can export the emissions of a specified radionuclide for use in subsequent calculations.

2.2.4 Federal Guidance Report 12

ICRP Publications 30 and 60 provide dose coefficients for the ingestion and inhalation intake of radionuclides. Dose coefficients for exposure to the radiations emitted by radionuclides present outside the body are given in Federal Guidance Report 12. That report addresses radionuclides uniformly distributed in air, in water, on the surface of the soil and within the volume of the soil. The published report is consistent with ICRP Publication 26 however the CD Supplement to Federal Guidance Report 13 provides values for the effective dose as defined in ICRP Publication 60.

2.2.5 When To Use "+E" DCCs

Several of the isotopes are listed with a '+E' designation. This designation indicates that the dose coefficient includes the contribution from ingrowth of daughter isotopes out to 1000 years. The intention of this designation is to make realistic DCCs by including the contributions from their short-lived decay products. The +E designation indicates that the parent radionuclide is accompanied by its progeny. In general, the activity of chain members assumes progeny ingrowth of 1000 years. There is one exception for the inhalation dose coefficient for Rn-222+E. EPA assumes a 50% equilibrium value for radon decay products (Po-218, Pb-214, Bi-214 and Po-214) in air. Before applying DCCs to a site, it should be determined if the isotopes present are in secular equilibrium. If the isotopes are found to be in secular equilibrium, the +E DCCs should be used for the parent isotope and the daughters included in the +E can be ignored. If the isotopes are not in secular equilibrium, DCCs should be applied individually for each daughter isotope. However, in the absence of empirical data, the "+E" values for radionuclides should be used unless there are compelling reasons not to.

For example, if analytical data from a site reveal that Ra-226, Rn-222 and Po-218 are detected at a site and that they are in secular equilibrium, the DCC for Ra-226+E should be applied and the Rn-222 and Po-218 can be ignored.

Another example could concern a decay chain in secular equilibrium like Th-228. Even though the decay chain for Th-228 is long, there is no Th-228+E dose coefficient because the activity of any of the progeny after 100 years of chain ingrowth was less than 90% of the parent's activity. The general rule for non-branching chains is, if a parent has any progeny with greater than 90% of the parent's activity at 1000 years, a +E is calculated. For branching chains, if any branch contributes over 90% of the branch fraction activity, at 1000 years, a +E is calculated. An example of this would be Cs-137+E where the branching fraction is 94.4 % to Ba-137m and at 100 years Ba-137m contributes over 90% of the activity than the parent, Cs-137. In either chain type, if the total dose contribution of the progeny is insignificant relative to the parent's contribution, then no +E values are calculated.

In this case of Th-228, the DCCs for Th-228, Ra-224+E, Po-216, Pb-212, Bi-212+E and Tl-208 should be used. If no part of the decay chain is in secular equilibrium, the user should use each of the DCCs for isotopes in the decay chain that have dose coefficients (e.g., Ra-224, Rn-220, Po-216, Pb-212, Bi-212, Po-212, and Tl-208). If part of the decay chain is in secular equilibrium, then the user may use that particular +E dose coefficient that covers that part of the decay chain, while using the dose coefficients for the other radionuclides.

2.2.6 Associated Decay Chains for "+D" and "+E" DCCs

Selected radionuclides and radioactive decay chain products are designated with the suffix "+D" or "+E" to indicate that dose estimates for these radionuclides include the contributions from their short-lived decay products, assuming equal activity concentrations (i.e., secular equilibrium) with the principal or parent nuclide in the environment. For all radionuclides without the "+D" or "+E" suffix, only intake or external exposure to the single radionuclide is considered. Most radionuclides with a "+D" or "+E" designation include the entire decay chain to the stable terminal nuclide in the dose concentration factors. The [Center for Radiation Protection Knowledge](#) provides tables of the 100 year and 1000 year progeny ingrowth for the isotopes in Table 2 and Table 3, respectively, of Appendix A of [Calculations of Slope Factors and Dose Coefficients](#). This table provides the associated decay chain used in the dose coefficients. These tables are reproduced below for common isotopes with the terminal nuclide added. The terminal nuclide is not included in the +E decay chain; it represents either the stable product or the next nuclide in the decay series that should be evaluated in addition to the principal radionuclide.

Principal Radionuclide (half-life in years)	Associated decay chain	Terminal Radionuclide	Half-life (yr)
Cs-137+D (30.2)	Ba-137m	Ba-137	stable
Cs-137+E (30.2)	Ba-137m	Ba-137	stable
Pu-239+D (2.41E+04)	U-235m	U-235	7.04E+08
Pu-239+E (2.41E+04)	U-235m	U-235	7.04E+08
Ra-226+D (1.60E+03)	Rn-222, Po-218, Pb-214, At-218, Bi-214, Rn-218, Po-214, Tl-210	Pb-210	22
Ra-226+E (1.60E+03)	Rn-222, Po-218, Pb-214, At-218, Bi-214, Rn-218, Po-214, Tl-210	Pb-210	22
Ra-228+D (5.75E+00)	Ac-228	Th-228	2
Ra-228+E (5.75E+00)	Ac-228	Th-228	2
Rn-222+D (1.05E-02)	Po-218	Pb-214	5.10E-05
Rn-222+E (1.05E-02)	Po-218	Pb-214	5.10E-05
Sr-90+D (28.8)	Y-90	Zr-90	stable
Sr-90+E (28.8)	Y-90	Zr-90	stable
Th-232+D (1.41E+10)	Ra-228, Ac-228	Th-228	1.91E+00
Th-232+E (1.41E+10)	Ra-228, Ac-228, Th-228, Ra-224, Rn-220, Po-216, Pb-212, Bi-212, Po-212, Tl-208	Pb-208	stable
U-235+D (7.04E+08)	Th-231	Pa-231	3.3E+04
U-235+E (7.04E+08)	Th-231	Pa-231	3.3E+04
U-238+D (4.47E+09)	Th-234, Pa-234m, Pa-234	U-234	2.4E+05
U-238+E (4.47E+09)	Th-234, Pa-234m, Pa-234	U-234	2.4E+05

2.2.7 Metastable Isotopes

Most dose and risk coefficients are presented for radionuclides in their ground state. In the decay process, the newly formed nucleus may be in an excited state and emit radiation; e.g., gamma rays, to lose the energy of the state. The excited nucleus is said to be in a metastable state which is denoted by the chemical symbol and atomic number appended by "m"; e.g., Ba-137m. If additional higher energy metastable states are present then "n", "p", ... is appended. Metastable states have different physical half-lives and emit different radiations and thus unique dose and risk coefficients. In decay data tabulations of [ICRP 107](#), if the half-life of a metastable state was less than 1 minute then the radiations emitted in de-excitation are included with those of the parent radionuclide. Click to see a graphical representation of the decay of [Cs-137 to Ba-137](#).

Eu-152, in addition to its ground state has two metastable states: Eu-152m and Eu-152n. The half-lives of Eu-152, Eu-152m and Eu-152n are: 13.5 y, 9.31 min and 96 min, respectively and the energy emitted per decay is 1.30 MeV, 0.080 MeV, and 0.14 MeV, respectively.

2.3 BDCC in Context of Superfund Modeling Framework

This BDCC calculator focuses on the application of a generic and simple site-specific approach that is part of a larger framework for calculating concentration levels for complying with dose based ARARs. Generic BDCCs for a 1 mrem/yr standard are provided by viewing either the tables in the [Download Area](#) section of this calculator or by running the [BDCC Search](#) section of this calculator with the "Get Default ARAR Concentrations" option.

Generic BDCCs are calculated from the same equations presented in the site-specific portion of the calculator, but are based on a number of default assumptions chosen to be protective of human health for most site conditions. Generic BDCCs, which should be scaled to the same dose level as the standard being complied (e.g., multiplied by a factor of ten for a 10 mrem/yr standard) can be used in place of site-specific BDCC levels; however, in general, they are expected to be more conservative than site-specific levels. The site manager should weigh the cost of collecting the data necessary to develop site-specific BDCCs with the potential for deriving a higher BDCC that provides an appropriate level of protection.

To avoid unnecessary inconsistency between radiological and chemical risk assessment and radiological dose assessment at the same site, users should generally use the same model for chemical and radionuclide risk assessment and radionuclide dose assessment. If there is a reason, on a site-specific basis, for using another model, then justification for doing so should be developed. The justification should include specific supporting data and information in the administrative record. The justification would normally include the model runs, using both the recommended EPA BDCC model and the alternative model. Users are cautioned that they should have a thorough understanding of both the BDCC recommended model and any alternative model when evaluating whether a different approach is appropriate. When alternative models are used, the user should adjust the default input parameters to be as close as possible to the BDCC inputs, which may be difficult since models tend to use different definitions for parameters. Numerous computerized mathematical models have been developed by EPA and other organizations to predict the fate and transport of radionuclides in the environment; these models include single-media unsaturated zone models (for example, groundwater transport) as well as multi-media models. These models have been designed for a variety of goals, objectives, and applications; as such, no single model may be appropriate for all site-specific conditions. Generally, even when a different model is used to predict fate and transport of radionuclides through different media, EPA recommends using the BDCC calculators for the remedial program to establish the dose-based concentrations to ensure consistency with CERCLA, the NCP, and EPA's Superfund guidance for remedial sites. Prior to using another model for dose assessment at a CERCLA remedial site, EPA regional staff should consult with the Superfund remedial program's National Radiation Expert (Stuart Walker, at (703) 603-8748 or walker.stuart@epa.gov. For more information on this issue, please see questions 10 and 16 on pages 12, 17, and 18 of [Radiation Risk Assessment at CERCLA Sites: Q & A](#) (EPA 540-R-012-13, May 2014).

3. Using the BDCC Table

The recommended BDCC "[Download Area](#)" tables provides generic recommended concentrations in the absence of site-specific exposure assessments. Screening concentrations can be used for:

- Prioritizing multiple sites within a facility
- Setting risk-based detection limits for contaminants of potential concern (COPCs)
- Focusing further assessment or response actions for the site or building

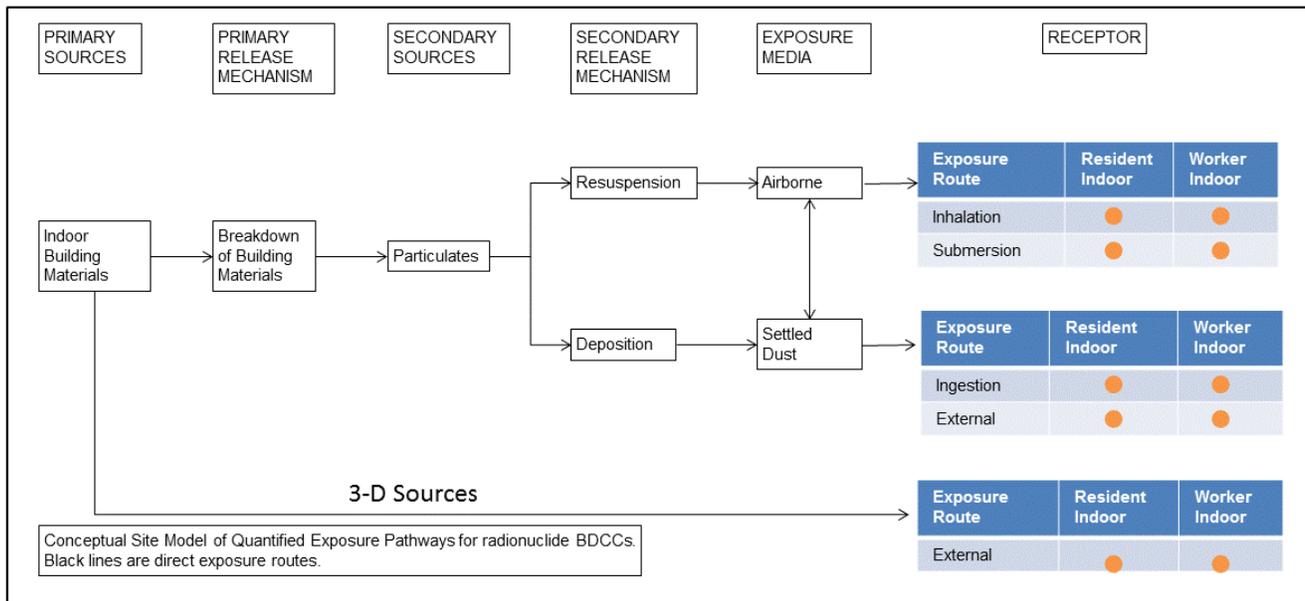
3.1 Developing a Conceptual Site Model

When using these recommended BDCCs, the exposure pathways of concern and site conditions should match those taken into account by the screening levels. Thus, normally, a conceptual site model (CSM) should be developed to identify likely contaminant source areas, exposure pathways, and potential receptors. This information can be used to determine the applicability of screening levels at the site and the need for additional information. The final CSM should represent linkages among contaminant sources, release mechanisms, exposure pathways, and routes and receptors based on historical information. Exposure routes could include: ingestion, inhalation, external exposure or submersion. See Section 4.3.9 for the consideration of dermal exposure. A CSM should summarize the understanding of the contamination problem.

Existing EPA documents with additional CSM guidance are:

1. [Risk Assessment Guidance for Superfund: Volume I Human Health Evaluation Manual \(Part D. Standardized Planning, Reporting, and Review of Superfund Risk Assessments\)](#). See Planning Table 1; and
2. [Soil Screening Guidance for Radionuclides: User's Guide](#). See Attachment A.

CSMs may be tabular, graphical or stem-and-leaf. Section 4 of the user guide presents links to graphical CSMs for each scenario. Below is a stem-and-leaf CSM showing the exposure routes quantified and not quantified in this calculator.



As a final check, the CSM should answer the following questions:

- Are there potential ecological concerns?
- Is there potential for other uses of the building other than those covered by the recommended BDCC levels (e.g., agricultural, recreational or trespasser)?
- Are there other likely human exposure pathways that were not considered in development of the recommended BDCC levels?
- Are there unusual site conditions?

The recommended BDCCs may need to be adjusted to reflect the answers to these questions, and additional tools or assessment methodologies may need to be considered (e.g., if there may be potentially significant ecological risk). The recommended default scenarios in this calculator are based the same default scenarios EPA provides in its guidance. Other scenarios may be investigated, using the BDCC calculator, by altering recommended site-specific exposure parameters.

3.2 Radionuclide Background

Natural background radiation should be considered prior to applying the recommended BDCCs to develop cleanup levels. Background and site-related levels of radiation should be addressed as they are for other hazardous substances, pollutants and contaminants at CERCLA sites. For further information, see EPA's guidance "[Role of Background in the CERCLA Cleanup Program](#)", April 2002, (OSWER 9285.6-07P). It should be noted that certain ARARs may specifically address how to factor background into cleanup levels. For example, some radiation ARAR levels are established as increments above background concentrations. In these circumstances, background should be addressed in the manner prescribed by the ARAR. Additional information on radioactive materials present in building materials can be found in [Volume 105, Number 2, March-April 2000, Journal of Research of the National Institute of Standards and Technology, Radioactivity Measurements on Glazed Ceramic Surfaces](#).

3.3 Potential Problems

As with any risk/dose-based tool, the potential exists for misapplication. In most cases, this results from not understanding the intended use of the recommended BDCCs. In order to prevent misuse of the recommended BDCCs, the following should be avoided:

- Applying recommended BDCC levels to a site without adequately developing a conceptual site model that identifies relevant exposure pathways and exposure scenarios
- Use of recommended BDCC levels as cleanup levels without the consideration of other relevant criteria
- Use of recommended BDCC levels as cleanup levels without verifying numbers with a health physicist/risk assessor
- Use of outdated recommended BDCC tables that have been superseded by more recent publications
- Not considering the effects from the presence of multiple isotopes

4. Land Use Descriptions, Equations, and Technical Documentation

The recommended BDCCs consider human exposure from direct contact with contaminated dust and air and external exposure to contaminated building materials. The equations and technical discussion are aimed at developing concentration levels for risk-based cleanup. Calculation of the recommended BDCCs are based on the [BDCC Calculator](#). The following text presents the recommended land use equations and their exposure routes. Table 1 presents the recommended definitions of the variables and their default values. Any alternative values or assumptions used in remedy evaluation or selection on a CERCLA site should be presented with supporting rationale in the Administrative Record.

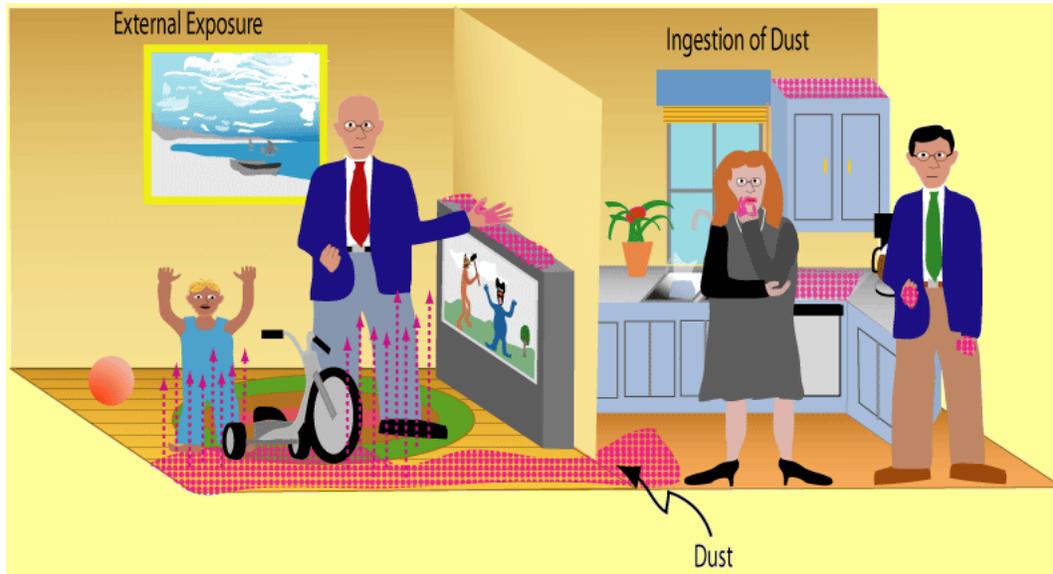
4.1 Resident

The recommended BDCC equations for the residential exposure scenario, presented here, contain the following exposure pathways and exposure routes

4.1.1 Exposure to Settled Dust on Surfaces

The resident is exposed to the radioactive contaminants in dust that settles in the building. Exposure is via two exposure routes. The first exposure route is external exposure. The second exposure route is ingestion. Ingestion of dust occurs when hands contact dust-laden surface and then come in contact with the mouth. Variation is allowed for contact with hard and soft surfaces as the transfer to skin varies on surface type.

Graphical Representation



Equations

Ingestion:

$$BDCC_{res_dust_ing} \left(\frac{pCi}{cm^2} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(\frac{1 - e^{-kt_{res}}}{kt_{res}} \right) \times \left(1 - e^{-\lambda t_{res}} \right) \times DCF_o \left(\frac{mrem}{pCi} \right) \times IFD_{res-adj} \left(\frac{123,025 \text{ cm}^2}{year} \right) \times F_{in} \times F_i}$$

where:

$$IFD_{res-adj} \left(\frac{123,025 \text{ cm}^2}{year} \right) = \left[\left[\left(FTSS_h (0.5) \times EF_{res-c} \left(\frac{350 \text{ days}}{year} \right) \times ET_{res-c,h} \left(\frac{6 \text{ hours}}{day} \right) \right) + \left(FTSS_s (0.1) \times EF_{res-c} \left(\frac{350 \text{ days}}{year} \right) \times ET_{res-c,s} \left(\frac{10 \text{ hours}}{day} \right) \right) \right] \times \right. \\ \left. SE (0.5) \times AAF_{res-c} (0.23) \times SA_{res-c} \left(\frac{16 \text{ cm}^2}{event} \right) \times FQ_{res-c} \left(\frac{17 \text{ events}}{hour} \right) \right] + \\ \left[\left[\left(FTSS_h (0.5) \times EF_{res-a} \left(\frac{350 \text{ days}}{year} \right) \times ET_{res-a,h} \left(\frac{6 \text{ hours}}{day} \right) \right) + \left(FTSS_s (0.1) \times EF_{res-a} \left(\frac{350 \text{ days}}{year} \right) \times ET_{res-a,s} \left(\frac{10 \text{ hours}}{day} \right) \right) \right] \times \right. \\ \left. SE (0.5) \times AAF_{res-a} (0.77) \times SA_{res-a} \left(\frac{49 \text{ cm}^2}{event} \right) \times FQ_{res-a} \left(\frac{3 \text{ events}}{hour} \right) \right]$$

where:

$$AAF_{res-c} (0.23) = \left(\frac{ED_{res-c} (6 \text{ years})}{ED_{res} (26 \text{ years})} \right) \text{ and } AAF_{res-a} (0.77) = \left(\frac{ED_{res-a} (20 \text{ years})}{ED_{res} (26 \text{ years})} \right)$$

External:

$$BDCC_{res_dust_ext} \left(\frac{pCi}{cm^2} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(\frac{1 - e^{-kt_{res}}}{kt_{res}} \right) \times \left(1 - e^{-\lambda t_{res}} \right) \times DCF_{ext-gp} \left(\frac{mrem/year}{pCi/cm^2} \right) \times F_{in} \times F_i \times F_{AM} \times F_{OFF-SET} \times EF_{res} \left(\frac{350 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Total:

$$BDCC_{res_dust_tot} \left(pCi/cm^2 \right) = \frac{1}{\frac{1}{BDCC_{res_dust_ing}} + \frac{1}{BDCC_{res_dust_ext}}}$$

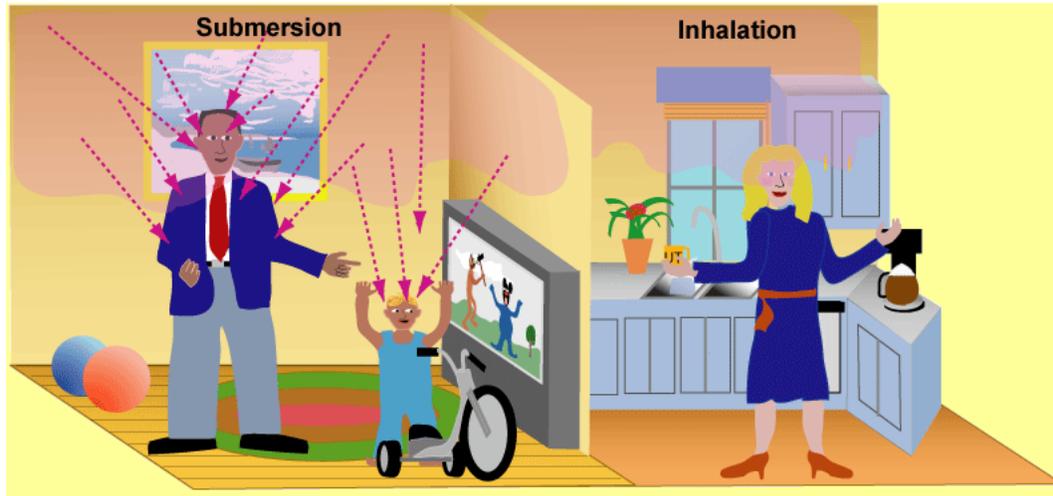
The resulting units for this recommended BDCC are in pCi/cm². The units are based on area because the SF used is the ground plane for external exposure and the ingestion route is based on hand surface area contacting dust on surfaces and subsequent hand to mouth transfer events. This equation is for values of k that are greater than 0; when k=0, the dissipation term is not quantified to avoid division by zero.

4.1.2 Exposure to Ambient Air with half-life decay

Ambient air exposure equations are presented below. These equations include a half-life decay function. In situations where the contaminant in the air is not being replenished (e.g., contaminated settled dust from a previous release that is being resuspended), these equations should be used.

The resident is exposed to the air in the home via two exposure routes. The first exposure route is inhalation of air. Inhalation is assumed to occur for the entire 24 hour day. The second exposure route is submersion. Submersion is external exposure from the contaminated air.

Graphical Representation



Equations

Inhalation:

$$BDCC_{res_air_decay_inh} \left(\frac{pCi}{m^3} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{res}} \right) \times DCF_i \left(\frac{mrem}{pCi} \right) \times IFA_{res-adj} \left(\frac{6,195 \text{ m}^3}{year} \right) \times F_{in} \times F_i}$$

where:

$$IFA_{res-adj} \left(\frac{6,195 \text{ m}^3}{year} \right) = \left[\begin{array}{l} \left[IRA_{res-c} \left(\frac{10 \text{ m}^3}{day} \right) \times ET_{res-c} \left(\frac{24 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{res-c} \left(\frac{350 \text{ days}}{year} \right) \times AAF_{res-c} (0.23) \right] + \\ \left[IRA_{res-a} \left(\frac{20 \text{ m}^3}{day} \right) \times ET_{res-a} \left(\frac{24 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{res-a} \left(\frac{350 \text{ days}}{year} \right) \times AAF_{res-a} (0.77) \right] \end{array} \right]$$

where:

$$AAF_{res-c} (0.23) = \left(\frac{ED_{res-c} (6 \text{ years})}{ED_{res} (26 \text{ years})} \right) \text{ and } AAF_{res-a} (0.77) = \left(\frac{ED_{res-a} (20 \text{ years})}{ED_{res} (26 \text{ years})} \right)$$

Submersion:

$$BDCC_{res_air_decay_sub} \left(\frac{pCi}{m^3} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{res}} \right) \times DCF_{sub} \left(\frac{mrem/year}{pCi/m^3} \right) \times GSF_a \times F_{in} \times F_i \times ET_{res} \left(\frac{24 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{res} \left(\frac{350 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ day}} \right)}$$

Total:

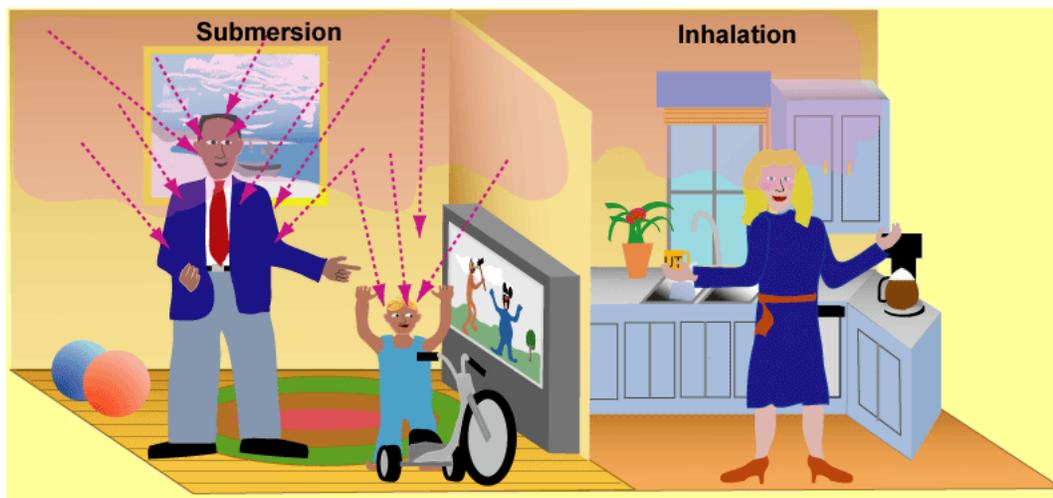
$$BDCC_{res_air_decay_tot} \left(\frac{pCi/m^3} \right) = \frac{1}{\frac{1}{BDCC_{res_air_decay_inh}} + \frac{1}{BDCC_{res_air_decay_sub}}}$$

4.1.3 Exposure to Ambient Air without half-life decay

Ambient air exposure equations are presented below. These equations do not include a half-life decay function. In situations where the contaminant in the air has a continual source (e.g., indoor radon from radium in the soil), these equations should be used.

The resident is exposed to the air in the home via two exposure routes. The first exposure route is inhalation of air. Inhalation is assumed to occur for the entire 24 hour day. The second exposure route is submersion. Submersion is external exposure from the contaminated air.

Graphical Representation



Equations

Inhalation:

$$BDCC_{res-air-nodecay-inh} \left(\frac{pCi}{m^3} \right) = \frac{DL \left(\frac{mrem}{year} \right)}{DCF_i \left(\frac{mrem}{pCi} \right) \times IFA_{res-adj} \left(\frac{6,195 m^3}{year} \right) \times F_{in} \times F_i}$$

where:

$$IFA_{res-adj} \left(\frac{6,195 m^3}{year} \right) = \left[IRA_{res-c} \left(\frac{10 m^3}{day} \right) \times ET_{res-c} \left(\frac{24 hours}{day} \right) \times \left(\frac{1 day}{24 hours} \right) \times EF_{res-c} \left(\frac{350 days}{year} \right) \times AAF_{res-c} (0.23) \right] + \left[IRA_{res-a} \left(\frac{20 m^3}{day} \right) \times ET_{res-a} \left(\frac{24 hours}{day} \right) \times \left(\frac{1 day}{24 hours} \right) \times EF_{res-a} \left(\frac{350 days}{year} \right) \times AAF_{res-a} (0.77) \right]$$

where:

$$AAF_{res-c} (0.23) = \left(\frac{ED_{res-c} (6 years)}{ED_{res} (26 years)} \right) \text{ and } AAF_{res-a} (0.77) = \left(\frac{ED_{res-a} (20 years)}{ED_{res} (26 years)} \right)$$

Submersion:

$$BDCC_{res-air-nodecay-sub} \left(\frac{pCi}{m^3} \right) = \frac{DL \left(\frac{mrem}{year} \right)}{DCF_{sub} \left(\frac{mrem/year}{pCi/m^3} \right) \times GSF_a \times F_{in} \times F_i \times ET_{res} \left(\frac{24 hours}{day} \right) \times \left(\frac{1 day}{24 hours} \right) \times EF_{res} \left(\frac{350 days}{year} \right) \times \left(\frac{1 year}{365 days} \right)}$$

Total:

$$BDCC_{res-air-nodecay-tot} \left(pCi/m^3 \right) = \frac{1}{\frac{1}{BDCC_{res-air-nodecay-inh}} + \frac{1}{BDCC_{res-air-nodecay-sub}}}$$

4.1.4 3-D Direct External Exposure

The resident is exposed to the radioactive contaminants in the building materials of the walls, floor, and ceiling.

Graphical Representation

Resident - Direct External Exposure. Direct external exposure from these contaminants is the only exposure route in this scenario. This scenario uses soil volume slope factors.



Resident - 3-D Direct External Exposure to Settled Dust. Direct external exposure from these contaminants in the dust is the only exposure route in this scenario. This scenario uses ground plane slope factors.



Equations

Contaminated building materials in walls, floor and ceiling using soil volume toxicity values

$$BDCC_{res-3D-ext-sv} \left(\frac{pCi}{g} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (year) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{res}} \right) \times DCF_{ext-sv} \left(\frac{mrem/year}{pCi/g} \right) \times GSF_b \times F_{in} \times F_i \times F_{AM} \times F_{OFF-SET} \times F_{r-surf^{sv}} \times ET_{res} \left(\frac{24 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{res} \left(\frac{350 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Contaminated building materials in walls, floor and ceiling using 1cm soil volume toxicity values

$$BDCC_{res-3D-ext-1cm} \left(\frac{pCi}{g} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (year) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{res}} \right) \times DCF_{ext-1cm} \left(\frac{mrem/year}{pCi/g} \right) \times GSF_b \times F_{in} \times F_i \times F_{AM} \times F_{OFF-SET} \times F_{r-surf^{1cm}} \times ET_{res} \left(\frac{24 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{res} \left(\frac{350 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Contaminated building materials in walls, floor and ceiling using 5cm soil volume toxicity values

$$BDCC_{res-3D-ext-5cm} \left(\frac{pCi}{g} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{res}} \right) \times DCF_{ext-5cm} \left(\frac{mrem/year}{pCi/g} \right) \times GSF_b \times F_{in} \times F_i \times F_{AM} \times F_{OFF-SET} \times F_{r-surf^{5cm}} \times ET_{res} \left(\frac{24 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{res} \left(\frac{350 \text{ days}}{years} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Contaminated building materials in walls, floor and ceiling using 15cm soil volume toxicity values

$$BDCC_{res-3D-ext-15cm} \left(\frac{pCi}{g} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{res}} \right) \times DCF_{ext-15cm} \left(\frac{mrem/year}{pCi/g} \right) \times GSF_b \times F_{in} \times F_I \times F_{AM} \times F_{OFF-SET} \times F_{r-surf}^{15cm} \times ET_{res} \left(\frac{24 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{res} \left(\frac{350 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Contaminated dust on walls, floor and ceiling using ground plane toxicity values

$$BDCC_{res-3D-ext-gp} \left(\frac{pCi}{cm^2} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{res} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{res}} \right) \times DCF_{d-ext} \left(\frac{mrem/year}{pCi/cm^2} \right) \times GSF_b \times F_{in} \times F_I \times F_{AM} \times F_{OFF-SET} \times F_{r-surf}^{gp} \times ET_{res} \left(\frac{24 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{res} \left(\frac{350 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

The resulting units for this recommended BDCC are in pCi/cm². The units are based on area because the SF used is the ground plane for external exposure and the ingestion route is based on hand surface area contacting dust on surfaces and subsequent hand to mouth transfer events.

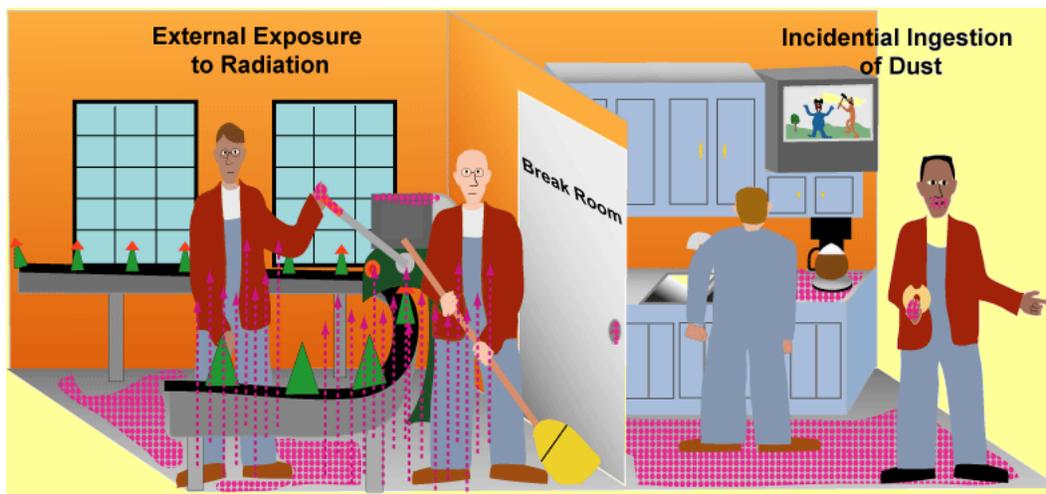
4.2 Indoor Worker

The recommended BDCC equations for the indoor worker exposure scenario, presented here, contain the following exposure pathways and exposure routes:

4.2.1 Exposure to Settled Dust on Surfaces

This worker is exposed to the radioactive contaminants in dust that settles in the building. Exposure is via two exposure routes. The first exposure route is external exposure. The second exposure route is ingestion. Ingestion of dust occurs when hands contact dust-laden surface and then come in contact with the mouth. Variation is allowed for contact with hard and soft surfaces as the transfer to skin varies on surface type.

Graphical Representation



Equations

Ingestion:

$$BDCC_{iw-dust-ing} \left(\frac{pCi}{cm^2} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(\frac{1 - e^{-k t_{iw}}}{k t_{iw}} \right) \times \left(1 - e^{-\lambda t_{iw}} \right) \times DCF_{oa} \left(\frac{mrem}{pCi} \right) \times IRD_{iw} \left(\frac{176.4 \text{ cm}^2}{day} \right) \times F_{in} \times F_I \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right)}$$

where:

$$IRD_{iw} \left(\frac{176.4 \text{ cm}^2}{day} \right) = \left[\left(FTSS_h (0.5) \times ET_{iw,h} \left(\frac{4 \text{ hours}}{day} \right) \right) + \left(FTSS_s (0.1) \times ET_{iw,s} \left(\frac{4 \text{ hours}}{day} \right) \right) \right] \times SE (0.5) \times SA_{iw} \left(\frac{49 \text{ cm}^2}{event} \right) \times FQ_{iw} \left(\frac{3 \text{ events}}{hour} \right)$$

External:

$$BDCC_{iw-dust-ext} \left(\frac{pCi}{cm^2} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(\frac{1-e^{-kt_{iw}}}{kt_{iw}} \right) \times \left(1-e^{-\lambda t_{iw}} \right) \times DCF_{ext-gp} \left(\frac{mrem/year}{pCi/cm^2} \right) \times F_{in} \times F_i \times F_{AM} \times F_{OFF-SET} \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Total:

$$BDCC_{iw-dust-tot} \left(pCi/cm^2 \right) = \frac{1}{\frac{1}{BDCC_{iw-dust-ing}} + \frac{1}{BDCC_{iw-dust-ext}}}$$

The resulting units for this recommended BDCC are in pCi/cm². The units are based on area because the SF used is the ground plane for external exposure and the ingestion route is based on hand surface area contacting dust on surfaces and subsequent hand to mouth transfer events. This equation is for values of k that are greater than 0; when k=0, the dissipation term is not quantified to avoid division by zero.

4.2.2 Exposure to Ambient Air with half-life decay

Ambient air exposure equations are presented below. These equations include a half-life decay function. They should be used in situations where the contaminant in the air is not being replenished (e.g., contaminated settled dust from a previous release that is being resuspended). This worker is exposed to the air in the building via two exposure routes. The first exposure route is inhalation of air. Inhalation is assumed to occur for the entire 8 hour work day. The second exposure route is submersion. Submersion is external exposure from the contaminated air.

Graphical Representation



Equations

Inhalation:

$$BDCC_{iw-air-decay-inh} \left(\frac{pCi}{m^3} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (year) \times \lambda \left(\frac{1}{year} \right)}{\left(1-e^{-\lambda t_{iw}} \right) \times DCF_1 \left(\frac{mrem}{pCi} \right) \times IRA_{iw} \left(\frac{60 \text{ m}^3}{day} \right) \times F_{in} \times F_i \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right)}$$

Submersion:

$$BDCC_{iw-air-decay-sub} \left(\frac{pCi}{m^3} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (year) \times \lambda \left(\frac{1}{year} \right)}{\left(1-e^{-\lambda t_{iw}} \right) \times DCF_{sub} \left(\frac{mrem/year}{pCi/m^3} \right) \times GSF_a \times F_{in} \times F_i \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Total:

$$BDCC_{iw-air-decay-tot} \left(pCi/m^2 \right) = \frac{1}{\frac{1}{BDCC_{iw-air-decay-inh}} + \frac{1}{BDCC_{iw-air-decay-sub}}}$$

4.2.3 Exposure to Ambient Air without half-life decay

Ambient air exposure equations are presented below. These equations do not include a half-life decay function. They should be used in situations where the contaminant in the air has a continual source (e.g., indoor radon from radium in the soil). This worker is exposed to the air in the building

via two exposure routes. The first exposure route is inhalation of air. Inhalation is assumed to occur for the entire 8 hour work day. The second exposure route is submersion. Submersion is external exposure from the contaminated air.

Graphical Representation



Equations

Inhalation:

$$BDCC_{iw-air-noddecay-inh} \left(\frac{pCi}{m^3} \right) = \frac{DL \left(\frac{mrem}{year} \right)}{DCF_I \left(\frac{mrem}{pCi} \right) \times IRA_{iw} \left(\frac{60 m^3}{day} \right) \times F_{in} \times F_i \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right)}$$

Submersion:

$$BDCC_{iw-air-noddecay-sub} \left(\frac{pCi}{m^3} \right) = \frac{DL \left(\frac{mrem}{year} \right)}{DCF_{sub} \left(\frac{mrem/year}{pCi/m^3} \right) \times GSF_a \times F_{in} \times F_i \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Total:

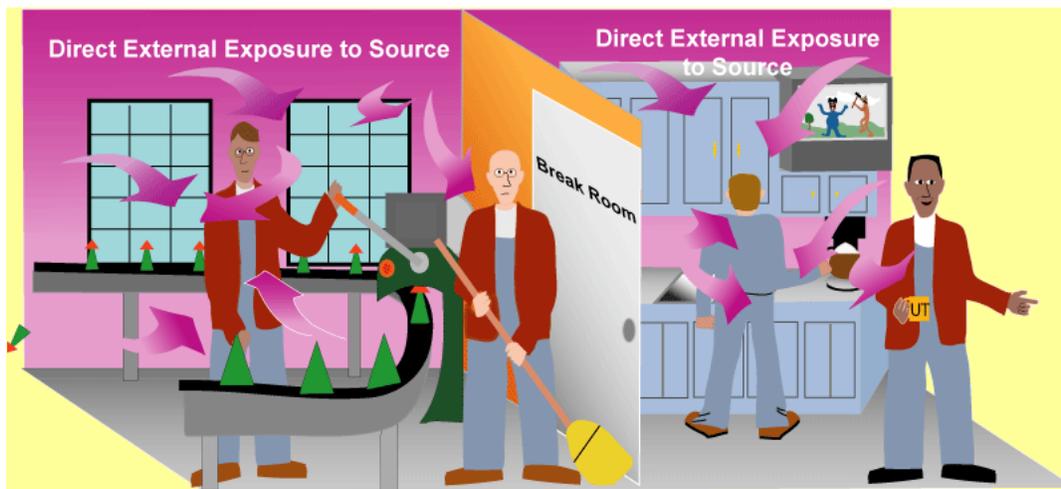
$$BDCC_{iw-air-noddecay-tot} \left(\frac{pCi/m^3} \right) = \frac{1}{\frac{1}{BDCC_{iw-air-noddecay-inh}} + \frac{1}{BDCC_{iw-air-noddecay-sub}}}$$

4.2.4 3-D Direct External Exposure

This worker is exposed to the radioactive contaminants in the building materials of the walls, floor, and ceiling.

Graphical Representation

Indoor Worker - Direct External Exposure. Direct external exposure from these contaminants is the only exposure route in this scenario. This scenario uses soil volume slope factors.



Resident - 3-D Direct External Exposure to Settled Dust. Direct external exposure from these contaminants in the dust is the only exposure route in this scenario. This scenario uses ground plane slope factors.



Equations

Contaminated building materials in walls, floor and ceiling using soil volume toxicity values

$$BDCC_{iw-3D-ext-sv} \left(\frac{pCi}{g} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{iw}} \right) \times DCF_{ext-sv} \left(\frac{mrem/year}{pCi/g} \right) \times GSF_b \times F_{in} \times F_1 \times F_{AM} \times F_{OFF-SET} \times F_{r-surf}^{sv} \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Contaminated building materials in walls, floor and ceiling using 1cm soil volume toxicity values

$$BDCC_{iw-3D-ext-1cm} \left(\frac{pCi}{g} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{iw}} \right) \times DCF_{ext-1cm} \left(\frac{mrem/year}{pCi/g} \right) \times GSF_b \times F_{in} \times F_1 \times F_{AM} \times F_{OFF-SET} \times F_{r-surf}^{1cm} \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Contaminated building materials in walls, floor and ceiling using 5cm soil volume toxicity values

$$BDCC_{iw-3D-ext-5cm} \left(\frac{pCi}{g} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{iw}} \right) \times DCF_{ext-5cm} \left(\frac{mrem/year}{pCi/g} \right) \times GSF_b \times F_{in} \times F_1 \times F_{AM} \times F_{OFF-SET} \times F_{r-surf}^{5cm} \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Contaminated building materials in walls, floor and ceiling using 15cm soil volume toxicity values

$$BDCC_{iw-3D-ext-15cm} \left(\frac{pCi}{g} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (1 \text{ year}) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{iw}} \right) \times DCF_{ext-15cm} \left(\frac{mrem/year}{pCi/g} \right) \times GSF_b \times F_{in} \times F_I \times F_{AM} \times F_{OFF-SET} \times F_{r-surf15cm} \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

Contaminated dust on walls, floor and ceiling using ground plane toxicity values

$$BDCC_{iw-3D-ext-gp} \left(\frac{pCi}{cm^2} \right) = \frac{DL \left(\frac{mrem}{year} \right) \times t_{iw} (year) \times \lambda \left(\frac{1}{year} \right)}{\left(1 - e^{-\lambda t_{iw}} \right) \times DCF_{ext-gp} \left(\frac{mrem/year}{pCi/cm^2} \right) \times GSF_b \times F_{in} \times F_I \times F_{AM} \times F_{OFF-SET} \times F_{r-surfgp} \times ET_{iw} \left(\frac{8 \text{ hours}}{day} \right) \times \left(\frac{1 \text{ day}}{24 \text{ hours}} \right) \times EF_{iw} \left(\frac{250 \text{ days}}{year} \right) \times \left(\frac{1 \text{ year}}{365 \text{ days}} \right)}$$

The resulting units for this recommended BDCC are in pCi/cm². The units are based on area because the SF used is the ground plane for external exposure and the ingestion route is based on hand surface area contacting dust on surfaces and subsequent hand to mouth transfer events.

4.3 Exposure Parameter Justification

The following sections describe the exposure parameter default variables and the values selected. The default parameter values are listed in Table 1.

4.3.1 Exposure Time (ET)

The exposure time represents the hours per day that a receptor spends exposed to a source. The exposure times vary by exposure scenario, age of the receptor and whether the source is located on a hard or soft surface. The EPA Office of Pesticide Programs (OPP) recommended defaults for a child resident are 8 hr/d for carpets and 4 hr/d for hard surfaces. Hard surface time is based on the estimated time spent in the kitchen and bathroom. Carpet time is based on remaining indoor time not including sleeping. This recommendation was judged to be representative of many children under age 6 who spend most of their time at home. After 18, many individuals will spend more time in school or at work. Others, however, may not work or attend school and spend more time at home. To be conservative, it was decided to recommend this second scenario and assume that adult residents would spend 8 hr/d on soft surfaces (carpet, sofa, etc.) and 4 hr/d on hard surfaces. This totals 12 hr/d. Assuming that an individual sleeps 8 hr/d, the total time in a residence is 20 hr/d. For this calculator, the remaining 4 hr/d were equally divided between exposure to hard and soft surfaces. This results in recommended default values of 10 hr/d on carpets and 6 hr/d on hard surfaces for adult and child residents. Ingestion of settled dust while sleeping is considered negligible because dust doesn't collect between sheets. Note, that inhalation and subsequent ingestion of dust particles trapped in mucus is not quantified in the recommended BDCC equations due to lack of exposure information. However, exposure to ambient air and direct external exposure continues during sleep. Additional dust ingestion may occur during food preparation or storage on hard surfaces but is not quantified in the recommended BDCC equations due to lack of exposure information.

For the indoor worker, exposure time for the dust ingestion exposure route is also divided between exposure to hard and soft surfaces. For this calculator, the recommended defaults were set at 4 hr/d for hard and soft surfaces. The recommended exposure time for exposure to ambient air and direct external exposure is the entire work day, or 8 hr/d.

4.3.2 Fraction Transferred from Surface to Skin (FTSS)

In general, this is the fraction of residue on a surface that can be transferred to skin. US EPA 2003 (pg D-5) states that hand press experiments were conducted on dry skin. Transfers of 10% were observed for carpets and 50% were observed for hard surfaces. These recommendations are considered representative of the WTC situation and were adopted for this calculator.

4.3.3 Surface Area (SA)

In general, this is the skin area contacted during the mouthing event. The OPP recommended default was based on the surface area of the 3 fingers that a child will most likely use for hand to mouth transfer. It was assumed that 3 fingers of one hand represents about 5% of the total area of both hands (EPA 2003). The exposure factor handbook (EPA 2011 Table 7.2) presents hand surface areas for adults and children. For children the surface areas were time-weight averaged across all age groups from birth to 6 years (317 cm²) and the 5% assumption was applied to derive the child hand surface area of 16 cm².

The hand surface area for the adult was also derived from data presented in the exposure factor handbook (EPA 2011 Table 7.2). The exposure factor handbook presents hand surface areas for adult's male and females of 1070 and 890 cm², respectively. These numbers were averaged (980 cm²) and the 5% assumption was applied to derive the adult hand surface area of 49 cm².

4.3.4 Frequency of Hand to Mouth (FQ)

The exposure factors handbook (EPA 2011 Table 4-1) and the world trade center report (EPA 2003) provide hand to mouth contact rates for many age groups. For the child FQ all age groups for mean indoor contact from birth to 6 years old were time-weight averaged from the exposure factor handbook. Missing data points were substituted with data from the nearest age group. The FQ for children was determined to be 17 times/hr.

For the adult FQ all age groups for mean indoor contact from 6 to 26 years old were time-weight averaged from the exposure factor handbook and world trade center report. The FQ for adults was determined to be 3 times/hr.

4.3.5 Saliva Extraction Factor (SE)

In general, the fraction transferred from skin to mouth will depend on the contaminant, mouthing time and other behavioral patterns. The OPP

recommended default is 50%, based on pesticide studies. Michaud et al (1994) assumed that all of the residues deposited on the fingertips would be transferred to the mouth, twice per day. In the Binghamton re-entry guideline derivation, a range of factors were used: 0.05, 0.1, and 0.25, representing the fraction of residue on hand that is transferred to the mouth (Kim and Hawley, 1985). For purposes of this assessment, the OPP recommended default of 50% was selected for all ages.

4.3.6 Age-Adjusted Dust Ingestion Rate (IF)

To account for the variability in exposure activities between children and adults, a recommended age-adjusted dust ingestion rate equation was developed. This equation is designed to take into account the differences in hand to mouth behavior, hand surface area, and exposure to hard and soft surfaces over the exposure durations of an adult and child.

4.3.7 Dust Ingestion Rate (IR_d)

To account for the variability in exposure to hard and soft surfaces, a recommended dust ingestion rate equation was developed. This equation averages the differences in exposure to hard and soft surfaces by the exposure times.

4.3.8 Dissipation Rate Constant (k)

In some circumstances, the load of dust on a contaminated surface, to which receptors are exposed, may decline over time. Dissipation of dust may result from cleaning, and transfer to skin and clothing. Different surfaces may be cleaned at different rates and any dissipation rate used should consider a representative cleaning frequency. To determine whether dissipation is a factor at a given site, the site manager should establish whether a significant reservoir of contaminated dust is present. Such reservoirs may function as sources of dust and negate the impacts of dissipation mechanisms. In fact, indoor concentrations of contaminants may be enhanced above their original outdoor source levels after repeated transfer inside (Paustenbach, Long, et al). The recommended first step in identifying the presence of a reservoir is to examine site history. If a waste site was created through disposal, deposition or equipment leaks over an extended period of time, then the contaminant may have seeped deep into the surface. Porous surfaces such as cement or wood are also more likely to have subsurface contamination. When reservoirs are less likely to exist, such as at sites where contamination is the result of a single spill, dust cloud or event, it may be more important to account for dissipation of surface loads. For fixed contamination in building materials, or on material surfaces, in the 3-D equations, the dissipation term is not included as dissipation is not expected.

The recommended default value for the dissipation rate constant is 0.0. This assumes that a contaminant reservoir is present. However, the variable is adjustable in the recommended BDCC calculator. If a dissipation rate constant is used, it is generally assumed that the dust was deposited as a one-time event (i.e.; dust cloud). Also, if a dissipation rate is applied, it is assumed that it is applicable from the point in time the recommended BDCC is calculated into the future. The discussion below provides a review of the literature related to this issue and provides an alternative dissipation rate constant value. Site specific dissipation rate constants can be used. This equation is for values of k that are greater than 0; when k=0, the dissipation term is not quantified to avoid division by zero. See the following text.

Based on many studies presented in EPA 2003 (pg. D-5), there is strong support for considering dissipation in setting criteria for building clean-ups. A study of the Binghamton State Office Building found that dioxin has dissipated over time according to first order kinetics with a 20 to 22 month half-life. This dissipation is thought to occur from a combination of cleaning, resuspension and dilution with uncontaminated dust (and possibly some volatilization). These same physical dissipation processes would apply to other compounds addressed in this study as well. Therefore, the other compounds were assumed to dissipate at the same rate as dioxin. In summary, a 22 month half-life (dissipation rate constant of 0.38 yr⁻¹) was adopted. Exposures were calculated in a series of time steps where the residue level was assumed to dissipate according to first order kinetics:

$$CSL = CSL_{\text{initial}} e^{-kt}$$

CSL = Contaminant Surface Load (ug/cm²)

CSL_{initial} = Initial Contaminant Surface Load (ug/cm²)

k = Dissipation Rate Constant (1/yr)

t = Time (yr)

The above equation steps are shown for completeness. This recommended BDCC calculator computes a concentration of contaminants in dust that will not exceed a target risk. The equation above simply derives the amount of dust. For this recommended BDCC calculator, the only parts of the above equation that are relevant are the dissipation rate constant and time. By putting these variables in the denominator of the recommended BDCC resident and worker ingestion of dust equations, a higher recommended BDCC concentration would be calculated.

WARNING: Using a dissipation rate constant or changing the value of t should only be done once a complete understanding of the mathematics involved in deriving the equation is gained and the site conditions have been fully investigated. The following exhibits display the results obtained by changing the value t and k t is equal to ED in all equations.

In the simplified PRG equation: $PRG = TR / CDI * SF * (1 - e^{-kt}) / (kt)$, where PRG is preliminary remediation goal, TR is target risk, CDI is chronic daily intake, SF is the radionuclide-specific slope factor and $(1 - e^{-kt}) / (kt)$ is the dissipation term, [Exhibit 1](#) shows the results of changing t. [Exhibit 2](#) shows the results of changing k.

Exhibit 1. Results Obtained By Changing The Value t.

t	k	SF	CDI	TR	(1-e ^{-kt})/(kt)	PRG
yr	yr-1	risk/pCi	cm ²	risk	unitless	pCi/cm ²
0	0.38	1.00E-05	400	1.00E-06	1.00E+01	2.5E-04
1	0.38	1.00E-05	400	1.00E-06	8.32E-01	3.01E-04
2	0.38	1.00E-05	400	1.00E-06	7.00E-01	3.57E-04
3	0.38	1.00E-05	400	1.00E-06	5.97E-01	4.19E-04
4	0.38	1.00E-05	400	1.00E-06	5.14E-01	4.86E-04
5	0.38	1.00E-05	400	1.00E-06	4.48E-01	5.59E-04

6	0.38	1.00E-05	400	1.00E-06	3.94E-01	6.35E-04
7	0.38	1.00E-05	400	1.00E-06	3.50E-01	7.15E-04
8	0.38	1.00E-05	400	1.00E-06	3.13E-01	7.98E-04
9	0.38	1.00E-05	400	1.00E-06	2.83E-01	8.84E-04
10	0.38	1.00E-05	400	1.00E-06	2.57E-01	9.72E-04
11	0.38	1.00E-05	400	1.00E-06	2.36E-01	1.06E-03
12	0.38	1.00E-05	400	1.00E-06	2.17E-01	1.15E-03
13	0.38	1.00E-05	400	1.00E-06	2.01E-01	1.24E-03
14	0.38	1.00E-05	400	1.00E-06	1.87E-01	1.34E-03
15	0.38	1.00E-05	400	1.00E-06	1.75E-01	1.43E-03
16	0.38	1.00E-05	400	1.00E-06	1.64E-01	1.52E-03
17	0.38	1.00E-05	400	1.00E-06	1.55E-01	1.62E-03
18	0.38	1.00E-05	400	1.00E-06	1.46E-01	1.71E-03
19	0.38	1.00E-05	400	1.00E-06	1.38E-01	1.81E-03
20	0.38	1.00E-05	400	1.00E-06	1.32E-01	1.90E-03
21	0.38	1.00E-05	400	1.00E-06	1.25E-01	2.00E-03
22	0.38	1.00E-05	400	1.00E-06	1.20E-01	2.09E-03
23	0.38	1.00E-05	400	1.00E-06	1.14E-01	2.19E-03
24	0.38	1.00E-05	400	1.00E-06	1.10E-01	2.28E-03
25	0.38	1.00E-05	400	1.00E-06	1.05E-01	2.38E-03
26	0.38	1.00E-05	400	1.00E-06	1.01E-01	2.47E-03
27	0.38	1.00E-05	400	1.00E-06	9.75E-02	2.57E-03
28	0.38	1.00E-05	400	1.00E-06	9.40E-02	2.66E-03
29	0.38	1.00E-05	400	1.00E-06	9.07E-02	2.76E-03
30	0.38	1.00E-05	400	1.00E-06	8.77E-02	2.85E-03

Exhibit 2. Results Obtained By Changing The Value k.

t	k	SF	CDI	TR	(1-e ^{-kt})/(kt)	PRG
yr	yr-1	risk/pCi	cm ²	risk	unitless	pCi/cm ²
30	0.000001	1.00E-05	400	1.00E-06	1.00E+00	2.50E-04
30	0.033331	1.00E-05	400	1.00E-06	6.32E-01	3.95E-04
30	0.066661	1.00E-05	400	1.00E-06	4.32E-01	5.78E-04
30	0.099991	1.00E-05	400	1.00E-06	3.17E-01	7.89E-04
30	0.133321	1.00E-05	400	1.00E-06	2.45E-01	1.02E-03
30	0.166651	1.00E-05	400	1.00E-06	1.99E-01	1.26E-03
30	0.199981	1.00E-05	400	1.00E-06	1.66E-01	1.50E-03
30	0.233311	1.00E-05	400	1.00E-06	1.43E-01	1.75E-03
30	0.266641	1.00E-05	400	1.00E-06	1.25E-01	2.00E-03
30	0.299971	1.00E-05	400	1.00E-06	1.11E-01	2.25E-03
30	0.333301	1.00E-05	400	1.00E-06	1.00E-01	2.50E-03
30	0.366631	1.00E-05	400	1.00E-06	9.09E-02	2.75E-03
30	0.399961	1.00E-05	400	1.00E-06	8.33E-02	3.00E-03
30	0.433291	1.00E-05	400	1.00E-06	7.69E-02	3.25E-03
30	0.466621	1.00E-05	400	1.00E-06	7.14E-02	3.50E-03
30	0.499951	1.00E-05	400	1.00E-06	6.67E-02	3.75E-03
30	0.533281	1.00E-05	400	1.00E-06	6.25E-02	4.00E-03
30	0.566611	1.00E-05	400	1.00E-06	5.88E-02	4.25E-03
30	0.599941	1.00E-05	400	1.00E-06	5.56E-02	4.50E-03
30	0.633271	1.00E-05	400	1.00E-06	5.26E-02	4.75E-03
30	0.666601	1.00E-05	400	1.00E-06	5.00E-02	5.00E-03
30	0.699931	1.00E-05	400	1.00E-06	4.76E-02	5.25E-03
30	0.733261	1.00E-05	400	1.00E-06	4.55E-02	5.50E-03
30	0.766591	1.00E-05	400	1.00E-06	4.35E-02	5.75E-03
30	0.799921	1.00E-05	400	1.00E-06	4.17E-02	6.00E-03
30	0.833251	1.00E-05	400	1.00E-06	4.00E-02	6.25E-03
30	0.866581	1.00E-05	400	1.00E-06	3.85E-02	6.50E-03
30	0.899911	1.00E-05	400	1.00E-06	3.70E-02	6.75E-03
30	0.933241	1.00E-05	400	1.00E-06	3.57E-02	7.00E-03
30	0.966571	1.00E-05	400	1.00E-06	3.45E-02	7.25E-03
30	1	1.00E-05	400	1.00E-06	3.33E-02	7.50E-03

4.3.9 Dermal Exposure

Other possible exposure pathways that may be considered in a radiological analysis of a contaminated building would include internal contamination due to puncture wounds and dermal absorption of radionuclides deposited on the skin. However, the radiation doses caused by these two pathways are likely to be *de minimis* and much smaller than the doses caused by the other potential pathways already considered for most radionuclides (Kennedy and Strenge 1992 in Section 3.1.2). Therefore, dermal pathways are not included in the current version of this BDCC calculator. If one desires to calculate dermal risk, one method would be to calculate the dose based on either adherence of dust/soil to dry or wet skin. The mobility of the radionuclide, the range of the emitted beta particles, and the assumed exposure parameters may be used to determine the percentage contribution of each component to the total calculated risk. The partitioning coefficient (Kd) of the beta-emitting radionuclide of concern would be used to determine the significance of the sweat layer. If this value approaches zero, then contaminated soil particulates may dissolve, and diluted

concentrations should be estimated from the original soil concentrations. If K_d is greater than zero, then the range of the emitted beta particles is expected to become the most important factor in determining if the radionuclide yields an unacceptable dose. If the range exceeds the average distribution of the sweat layer, then risk calculations are likely warranted. The dry deposition scenario dominates the whole exposure interval. Otherwise, the radionuclide is shielded by the sweat layer, and the corresponding indirect deposition contributions to the total risk are negligible.

4.3.10 Room Surfaces Factor (F_{r-surf})

The 3-D direct external exposure equations (building materials and dust) without F_{r-surf} are single surface equations. The surfaces factor, in the recommended default and site-specific equations, is based on exposure to 4 walls, the floor and the ceiling in a room. This calculator uses the relationship between the dose rate coefficients for exposures in a contaminated room and dose rate coefficients for an infinite source to calculate a surfaces factor (F_{r-surf}). The dose quantity evaluated is the air kerma rate one meter above the floor. The floor, walls and ceiling of the rooms are assumed to be contaminated to the same level. In [Finklea 2015](#), 5 room sizes, ranging from 10 by 10 by 10 to 400 by 400 by 40 feet, were modeled to account for the dose contribution from multiple surfaces. Several individual materials including: wood, glass, concrete, drywall and adobe mud brick were analyzed as well as 2 composite scenarios where multiple building materials are present in different ratios. Composite 1 is a drywall room with a glass window, wooden doors and drywall walls. The floors for composite 1 are concrete and the ceiling is drywall. Composite 2 is a concrete room with wooden doors, a drywall ceiling and a concrete floor. Both composite cases used a homogeneous mix of material for the walls to represent the window and door mixed in with the wall. Four receptor positions were included for each material: average, center of room, corner of room and along the center of a wall. Contamination depths were considered to be surface, 1cm, 5cm, 15cm and 100cm (infinite). The F_{r-surf} for the default option is set to the most protective value across the 5 room sizes and 4 receptor positions. In the site-specific option the user can select from the 5 room sizes, 4 receptor positions and the building material. [Finklea 2015](#) presents the air kerma values and additional appendices were developed for this BDCC Calculator to give the ratios compared to infinite dose coefficients. The F_{r-surf} values for parent nuclides for adobe and concrete are given in [Appendix A and B](#). The F_{r-surf} values for parent nuclides for drywall, glass, and wood are given in [Appendices C, D, and E](#). The F_{r-surf} values for parent nuclides for room composite 1 are given in [Appendix E](#). The F_{r-surf} values for parent nuclides for room composite 2 are given in [Appendix G](#). The F_{r-surf} values for the "+D" nuclides are given in [Appendix H](#). The F_{r-surf} values for the "+E" nuclides are given in [Appendix I](#).

4.3.11 Radionuclide Decay Constant (lambda: λ)

Each equation (excluding air without decay) has a decay constant term (λ) which is based on the half-life of the isotope. The decay constant, λ , = $0.693/\text{half-life}$ in years (where, $0.693=\ln(2)$). The intention of this term is to derive realistic recommended BDCCs for isotopes with relatively short half-lives, compared to the exposure duration (ED). The term $(1 - e^{-\lambda t})$ takes into account the number of half-lives that will occur within the ED to calculate an appropriate value. Definitions of the input variables are in [Table 1](#).

4.3.12 Gamma Shielding Factors (GSF_a and GSF_b)

A shielding factor GSF_a was added to the air submersion equations with a default of 1 (no shielding). If the user has a site-specific shielding factor it can be applied in the calculator.

A shielding factor GSF_b was added to the 3-D building equations with a default of 1 (no shielding). If the user has a site-specific shielding factor it can be applied in the calculator.

4.4 Equation Sources and Parameters

This section presents details on some of the equation sources and parameters.

4.4.1 Exposure to Settled Dust on Surfaces Equations

Inadvertent ingestion from materials deposited on surfaces equation was modeled after the equation found in ANL 2001 (Fig 8.3). The ingestion rate term in this equation was modeled after EPA 2003 (pg. D-4).

External exposure from deposited materials equation was modeled after the equation found in ANL 2001 (Fig 8.7).

4.4.2 Ambient Air Exposure

The inhalation equation was modeled after the equation found in EPA 2003 (pg. D-1). The submersion equation was modeled after the equation found in ANL 2001 (Fig 8.1).

4.4.2.1 Submersion Pathway Equation Derivation

The air submersion external dose from exposure to indoor contaminated air was calculated by using the following equation:

$$D_{i,sub}^n(t) = F_{in} \times F_i \times C_i^n(t) \times DCF_{sub}^n$$

where:

F_{in} = fraction of time spent indoors

F_i = fraction of time spent in compartment i

$D_{i,sub}^n(t)$ = total annual air submersion effective dose equivalent from radionuclide n at time t in compartment i (mrem/yr);

$C_i^n(t)$ = average concentration of radionuclide n at time t in the indoor air of compartment i (3); and

$DCF_{sub}^n(t)$ = air submersion DCF for radionuclide n (mrem/yr per pCi/m³).

4.4.3 External Exposure

The direct external exposure from a volume and surface of a large area equation was modeled after ANL 2001 (Fig 8.6). External exposure from deposited materials equation was modeled after the equation found in ANL 2001 (Fig 8.7).

4.4.3.1 External Exposure Equation Derivation

The external exposure pathway dose from exposure to an area or a volume source containing radionuclide n in compartment i ,

D_w^n , is expressed as:

$$D_w^n = F_{in} \times F_i \times C_s^n \times DCF_v^n \times F_G^n,$$

where:

F_{in} = fraction of time spent indoors;

F_i = fraction of time spent in compartment i ;

C_s^n = average concentration of radionuclide n

DCF_v^n = FGR-12 dose conversion factor for infinite volume source; and

F_G^n = geometrical factor for finite area, source thickness, shielding, source material, and position of receptor relative to the source for radionuclide n .

The geometrical factor is the ratio of the effective dose equivalent for the actual source to the effective dose equivalent for the standard source. The standard source is a contaminated soil of infinite depth and lateral extent with no cover. The geometrical factor is expressed as the product of the depth-and-cover factor, F_{CD} , an area and material factor, F_{AM} , and the off-set factor, $F_{OFF-SET}$.

So,

F_G^n = effective dose from actual source/effective dose from standard source.

Then,

$$F_G^n = F_{CD} \times F_{AM} \times F_{OFF-SET}$$

4.4.3.1.1 Depth-And-Cover Factor (F_{CD})

Note: The F_{CD} would traditionally be included in this type of analysis, however, it is not included in the equations for this calculator. This calculator includes depth-specific dose conversion factors for surface (ground plane) and uniformly distributed volume sources at four specific thicknesses (1, 5, and 15 cm, and effectively infinite). Inclusion of these dose conversion factors eliminates the need for the F_{CD} .

Dose conversion factors in FGR-12 (Eckerman and Ryman 1993) are given for surface and uniformly distributed volume sources at four specific thicknesses (1, 5, and 15 cm, and effectively infinite) with a soil density of 1.6 g/cm³. FGR-12 assumes that sources are infinite in lateral extent. In actual situations, sources can have any depth, shape, cover, and size. A depth and-cover factor function, F_{CD} , was developed with regression analysis to express the attenuation for radionuclides. Three independent radionuclide-specific parameters were determined by using the effective dose equivalent values of FGR-12 at different depths. Kamboj et al. (1998) describes how the depth-and-cover function was derived using the effective dose equivalent values of FGR-12 at different depths. A depth-and-cover factor function was derived from the depth factor function by considering both dose contribution and attenuation from different depths:

$$\frac{D(T_c = t_c, T_s = t_s)}{D(T_c = 0, T_s = \infty)} = A e^{-K_A \rho_c t_c} (1 - e^{-K_A \rho_c t_s}) + B e^{-K_B \rho_c t_c} (1 - e^{-K_B \rho_c t_s}),$$

where:

A, B = fit parameters (dimensionless);

K_A, K_B = fit parameters (cm²/g);

t_c = shielding thickness (cm) (the sum of all shielding thicknesses between the source and the receptor), the shielding is placed immediately adjacent to the source;

ρ_c = shielding density (g/cm³) (the thickness-averaged density between the source and receptor);

t_s = source thickness (cm);

ρ_s = source density (g/cm³);

T_c = shielding parameter (m); and

T_s = source depth parameter (m).

The following constraints were put on the four fitting parameters:

1. All the parameters were forced to be positive;
2. $A + B = 1$; and
3. In the limit source depth, $t_s \rightarrow$ zero, the DCF should match the contaminated surface DCF.

All the four unknown parameters (A , B , K_A , and K_B) were found for 67 radionuclides available in the RESRAD-BUILD computer code. The fitted values of A , B , K_A , and K_B for radionuclides were used in the dose calculations.

4.4.3.1.2 Area-And-Material Factor (F_{AM})

For actual geometries (finite area and different materials), the area and material factor, F_{AM} , was derived by using the point-kernel method. This factor depends not only on the lateral extent of the contamination but also on source thickness, shielding thickness, gamma energies, and source material through its attenuation and buildup factors. All energies from radionuclide decay were considered separately and weighted by its yield, y , energy, E , and an energy dependent coefficient, K , to convert from air-absorbed dose to effective dose equivalent:

$$F_{AM} = \frac{\sum_{\text{Energies } j} y_j E_j K_j \int_V \frac{B(x) e^{-\mu x}}{(x')^2} dV'}{\sum_{\text{Energies } j} y_j E_j K_j \int_V \frac{B(x) e^{-\mu x}}{(x)^2} dV}$$

where:

$$(x')^2 = r^2 + (t_a + t_c + t)^2;$$

$$(x)^2 = r^2 + (1m + t)^2;$$

$$\mu = \frac{(t_a \mu_a + t_c \mu_c + t \mu_s)}{(t_a + t_c + t)}; \text{ and}$$

$$B(x) = B_a \left(\frac{t_a}{t_a + t_c + t_s} \chi \right) B_c \left(\frac{t_c}{t_a + t_c + t_s} \chi \right) B_s \left(\frac{t_s}{t_a + t_c + t_s} \chi \right).$$

B and μ are the buildup factor and the attenuation factor, respectively, for the appropriate material (a for air, c for shield material, and s for source material or soil reference). The integration volume V' is the desired geometry of specified material with radius R , shielding thickness t , and air thickness t_a ; whereas V is the reference geometry of soil extending infinitely laterally with no shield and the receptor midpoint located 1 m from the surface.

4.4.1.3 Off-set Factor ($F_{OFF-SET}$)

The off-set factor, $F_{OFF-SET}$, is the ratio of the dose estimates from a non-circular shaped contaminated material to a reference shape. The concept of the shape factor is used to calculate the off-set factor. The reference shape is a fully contaminated circular area encompassing the given shape, centered about the receptor. This factor is derived by considering the area, material factors of a series of concentric circles, and the corresponding contamination fraction of the annular regions. The off-set factor is obtained by enclosing the irregularly shaped contaminated area in a circle, multiplying the area factor of each annulus by the fraction of the contaminated annulus area, f , summing the products, and dividing by the area factor of a circular contaminated material that is equivalent in area:

$$F_{OFF-SET} = \frac{\sum_{i=0}^n f_i [F_{AM}(A_i) - F_A(A_{i-1})]}{F_{AM} \left[\sum_{i=0}^n f_i (A_i - A_{i-1}) \right]}$$

5. Recommended Default Exposure Parameters

Table 1 presents the definitions of the variables and their default values. The BDCC default values and exposure models are consistent with the Building Preliminary Remediation Goals for Radionuclides (BPRG) calculator. Both the BDCC and BPRG default values are consistent with the World Trade Center Assessment (EPA 2003) where the same pathways are addressed (e.g., ingestion and inhalation) and are analogous where pathways are similar (e.g., dermal and external exposure), except where the BDCC and BPRG have been updated to both follow the recommendations in the OSWER Directive concerning use of exposure parameters from the 2011 Exposure Factors Handbook. Any alternative values or assumptions used in remedy evaluation or selection on a CERCLA site should be

presented with supporting rationale in Administrative Records.

Table 1. Recommended Default Exposure Parameters

BDCC Equations			
Symbol ▼	Definition (units) <	Default <	Reference <
BDCC _{iw-3D-ext-15cm}	Indoor Worker 3-D Direct External Exposure (pCi/g)	Isotope-specific	Exposure to 15cm of contaminated dust on surfaces of building material. Developed for BDCC calculator.
BDCC _{iw-3D-ext-1cm}	Indoor Worker 3-D Direct External Exposure (pCi/g)	Isotope-specific	Exposure to 1cm of contaminated dust on surfaces of building material. Developed for BDCC calculator.
BDCC _{iw-3D-ext-5cm}	Indoor Worker 3-D Direct External Exposure (pCi/g)	Isotope-specific	Exposure to 5cm of contaminated dust on surfaces of building material. Developed for BDCC calculator.
BDCC _{iw-3D-ext-gp}	Indoor Worker 3-D Direct External Exposure (pCi/cm ²)	Isotope-specific	Exposure to ground plane contaminated dust on surfaces of building material. Developed for BDCC calculator.
BDCC _{iw-3D-ext-sv}	Indoor Worker 3-D Direct External Exposure (pCi/g)	Isotope-specific	Exposure to infinite depth soil volume on building material. Developed for BDCC calculator.
BDCC _{iw-air-decay-inh}	Indoor Worker Inhalation of Ambient Air (with half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator.
BDCC _{iw-air-decay-sub}	Indoor Worker Submersion in Ambient Air (with half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator.
BDCC _{iw-air-decay-tot}	Indoor Worker Total Exposure to Ambient Air (with half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator.
BDCC _{iw-air-nodecay-inh}	Indoor Worker Inhalation of Ambient Air (with no half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator. Exposure to ambient air.
BDCC _{iw-air-nodecay-sub}	Indoor Worker Submersion in Ambient Air (with no half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator. Exposure to ambient air.
BDCC _{iw-air-nodecay-tot}	Indoor Worker Total Exposure to Ambient Air (with no half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator. Exposure to ambient air.
BDCC _{iw-dust-ext}	Indoor Worker External Exposure to Settled Dust on Room Surfaces (pCi/cm ²)	Isotope-specific	Developed for BDCC calculator.
BDCC _{iw-dust-ing}	Indoor Worker Ingestion of Settled Dust on Room Surfaces (pCi/cm ²)	Isotope-specific	Developed for BDCC calculator.
BDCC _{iw-dust-tot}	Indoor Worker Total Exposure to Settled Dust on Room Surfaces (pCi/cm ²)	Isotope-specific	Developed for BDCC calculator.
BDCC _{res-3D-ext-15cm}	Resident 3-D Direct External Exposure (pCi/g)	Isotope-specific	Exposure to 15cm of contaminated dust on surfaces of building material. Developed for BDCC calculator.
BDCC _{res-3D-ext-1cm}	Resident 3-D Direct External Exposure (pCi/g)	Isotope-specific	Exposure to 1cm of contaminated dust on surfaces of building material. Developed for BDCC calculator.
BDCC _{res-3D-ext-5cm}	Resident 3-D Direct External Exposure (pCi/g)	Isotope-specific	Exposure to 5cm of contaminated dust on surfaces of building material. Developed for BDCC calculator.
BDCC _{res-3D-ext-gp}	Resident 3-D Direct External Exposure (pCi/cm ²)	Isotope-specific	Exposure to ground plane contaminated dust on surfaces of building material. Developed for BDCC calculator.
BDCC _{res-3D-ext-sv}	Resident 3-D Direct External Exposure (pCi/g)	Isotope-specific	Exposure to infinite depth soil volume on building material. Developed for BDCC calculator.
BDCC _{res-air-decay-inh}	Resident Inhalation of Ambient Air (with half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator.
BDCC _{res-air-decay-sub}	Resident Submersion in Ambient Air (with half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator.
BDCC _{res-air-decay-tot}	Resident Total Exposure to Ambient Air (with half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator.
BDCC _{res-air-nodecay-inh}	Resident Inhalation of Ambient Air (with no half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator. Exposure to ambient air.
BDCC _{res-air-nodecay-sub}	Resident Submersion in Ambient Air (with no half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator. Exposure to ambient air.
BDCC _{res-air-nodecay-tot}	Resident Total Exposure to Ambient Air (with no half-life decay) (pCi/m ³)	Isotope-specific	Developed for BDCC calculator. Exposure to ambient air.
BDCC _{res-dust-ext}	Resident External Exposure to Settled Dust on Room Surfaces (pCi/cm ²)	Isotope-specific	Developed for BDCC calculator.
BDCC _{res-dust-ing}	Resident Ingestion of Settled Dust on Room Surfaces (pCi/cm ²)	Isotope-specific	Developed for BDCC calculator.
BDCC _{res-dust-tot}	Resident Total Exposure to Settled Dust on Room Surfaces (pCi/cm ²)	Isotope-specific	Developed for BDCC calculator.
Dose Conversion Factors			
Symbol ▼	Definition (units) <	Default <	Reference <
DCF _{ext-15cm}	External Exposure Dose Conversion Factor - 15cm depth soil volume (mrem/yr per pCi/g)	Isotope-specific	ORNL 2014c
DCF _{ext-1cm}	External Exposure Dose Conversion Factor - 1cm depth soil volume (mrem/yr per pCi/g)	Isotope-specific	ORNL 2014c
DCF _{ext-5cm}	External Exposure Dose Conversion Factor - 5cm depth soil volume (mrem/yr per pCi/g)	Isotope-specific	ORNL 2014c

DCF _{ext-gp}	External Exposure Dose Conversion Factor - surface dust (mrem/yr per pCi/cm ²)	Isotope-specific	ORNL 2014c
DCF _{ext-sv}	External Exposure Dose Conversion Factor - infinite depth (mrem/yr per pCi/g)	Isotope-specific	ORNL 2014c
DCF _i	Inhalation Dose Conversion Factor - air (mrem/pCi)	Isotope-specific	ORNL 2014c
DCF _o	Dust Ingestion Dose Conversion Factor - population (mrem/pCi)	Isotope-specific	ORNL 2014c
DCF _{oa}	Dust Ingestion Dose Conversion Factor - adult only (mrem/pCi)	Isotope-specific	ORNL 2014c
DCF _{sub}	External Exposure Dose Conversion Factor - submersion (mrem/yr per pCi/m ³)	Isotope-specific	ORNL 2014c

Dose and Decay Constant Variables

Symbol ▼	Definition (units) <	Default <	Reference <
DL	Dose Limit (mrem/year)	1	User must specify dose limit
k	Dissipation Rate Constant - (yr ⁻¹)	0.0	EPA 2003 (pg. D-8)
t _{iw}	Time - worker (yr)	1	U.S. EPA 2014 (OSWER Directive 9200.1-120)
t _{res}	Time - resident (yr)	1	U.S. EPA 2014 (OSWER Directive 9200.1-120)
λ	decay constant = 0.693/half-life (year ⁻¹) where 0.693 = ln(2)	isotope-specific	Developed for Radionuclide Soil Screening Calculator (EPA 2000c)

Miscellaneous Variables

Symbol ▼	Definition (units) <	Default <	Reference <
AAF _{res-a}	Annual Age Fraction - resident adult (unitless)	0.77	This fraction is used to compose an age-adjusted intake.
AAF _{res-c}	Annual Age Fraction - resident child (unitless)	0.23	This fraction is used to compose an age-adjusted intake.
F _{AM}	Area and Material Factor (unitless)	1.0	ANL 2001 (Fig 8.6)
F _i	Fraction of time spent in compartment (unitless)	1.0	ANL 2001 (Fig 8.1)
F _{in}	Fraction time spent indoor (unitless)	1.0	ANL 2001 (Fig 8.1)
F _{OFF-SET}	Off-set Factor (unitless)	1.0	ANL 2001 (Fig 8.6)
FQ _{iw}	Frequency of Hand to Mouth - worker (event/hr)	3	EPA 2011 Table 4.1 and EPA 2003. Time weighted average of all age groups from 6 to 26 years.
FQ _{res-a}	Frequency of Hand to Mouth - adult (event/hr)	3	EPA 2011 Table 4.1 and EPA 2003. Time weighted average of all age groups from 6 to 26 years.
FQ _{res-c}	Frequency of Hand to Mouth - child(event/hr)	17	EPA 2011 Table 4.1 and EPA 2003. Time weighted average of all age groups from birth to 6 years.
F _{r-surf15cm}	Room Surfaces Factor for 15 cm Soil Volume (unitless)	Isotope-specific	Finklea 2015
F _{r-surf1cm}	Room Surfaces Factor for 1 cm Soil Volume (unitless)	Isotope-specific	Finklea 2015
F _{r-surf5cm}	Room Surfaces Factor for 5 cm Soil Volume (unitless)	Isotope-specific	Finklea 2015
F _{r-surfBP}	Room Surfaces Factor for Ground Plane (unitless)	Isotope-specific	Finklea 2015
F _{r-surfsv}	Room Surfaces Factor for Infinite Soil Volume (unitless)	Isotope-specific	Finklea 2015
FTSS _h	Fraction Transferred Surface to Skin - hard surface (unitless)	0.5	EPA 2003 (pg. D-3)
FTSS _s	Fraction Transferred Surface to Skin - soft surface (unitless)	0.1	EPA 2003 (pg. D-3)
GSF _a	Gamma Shielding Factor for air (unitless)	1 (assumes no shielding)	Other GSFs are presented in these reports. U.S. EPA 2000a. (pg. 2-22). U.S. EPA 2000b. (pg. 2-18)
GSF _b	Gamma Shielding Factor for building surfaces (unitless)	1 (assumes no shielding)	Other GSFs are presented in these reports. U.S. EPA 2000a. (pg. 2-22). U.S. EPA 2000b. (pg. 2-18)
SA _{iw}	Surface Area of Fingers - indoor worker (cm ² /event)	49	EPA 2011 Table 7.2. 5% of the average of adult male and female.
SA _{res-a}	Surface Area of Fingers - adult (cm ² /event)	49	EPA 2011 Table 7.2. 5% of the average of adult male and female.
SA _{res-c}	Surface Area of Fingers - child (cm ² /event)	16	EPA 2011 Table 7.2. 5% of the time weighted average of all age groups from birth to 6 years.
SE	Saliva Extraction Factor (unitless)	0.5	EPA 2003 (pg. D-5)

Inhalation and Ingestion Rates

Symbol ▼	Definition (units) <	Default <	Reference <
IFA _{res-adj}	Age-Adjusted Inhalation Fraction - resident (m ³ /year; based on IRIS default)	6,195	U.S. EPA 1991 (pg. 15)
IFD _{res-adj}	Age-Adjusted Dust Ingestion Fraction - resident (cm ² /year)	123,025	Calculated Value based on EPA 2003 (pg. D-4)
IRA _{iw}	Inhalation Rate - indoor worker (m ³ /day; based on a rate of 2.5m ³ /hr for 24hr)	60	U.S. EPA 1991 (pg. 15)

IRA _{res-a}	Inhalation Rate - resident adult (m ³ /day; based on IRIS default)	20	U.S. EPA 1991 (pg. 15)
IRA _{res-c}	Inhalation Rate - resident child (m ³ /day; based on IRIS default)	10	U.S. EPA 1997 (pg. 5-11)
IRD _{iw}	Dust Ingestion Rate - indoor worker (cm ² /day)	176.4	Calculated Value based on EPA 2003 (pg. D-4)

Exposure Frequency, Exposure Duration, and Exposure Time Variables

Symbol ▼	Definition (units) ▼	Default ▼	Reference ▼
ED _{res}	Exposure Duration - resident (yr)	26	U.S. EPA 2014 (OSWER Directive 9200.1-120)
ED _{res-a}	Exposure Duration - adult resident (yr)	20	U.S. EPA 2014 (OSWER Directive 9200.1-120)
ED _{res-c}	Exposure Duration - child resident (yr)	6	U.S. EPA 2014 (OSWER Directive 9200.1-120)
EF _{iw}	Exposure Frequency - worker (day/yr)	250	U.S. EPA 2014 (OSWER Directive 9200.1-120)
EF _{res}	Exposure Frequency - resident (day/yr)	350	U.S. EPA 2014 (OSWER Directive 9200.1-120)
EF _{res-a}	Exposure Frequency - resident adult (day/yr)	350	U.S. EPA 2014 (OSWER Directive 9200.1-120)
EF _{res-c}	Exposure Frequency - resident child (day/yr)	350	U.S. EPA 2014 (OSWER Directive 9200.1-120)
ET _{iw}	Air Exposure Time - indoor worker (hr/day)	8	U.S. EPA 2014 (OSWER Directive 9200.1-120)
ET _{iw,h}	Exposure Time - indoor worker hard surface (hr/day)	4	EPA 2003 (pg. D-4)
ET _{iw,s}	Exposure Time - indoor worker soft surface (hr/day)	4	EPA 2003 (pg. D-4)
ET _{res}	Air Exposure Time - resident (hr/day)	24	U.S. EPA 2014 (OSWER Directive 9200.1-120)
ET _{res-a}	Air Exposure Time - resident adult(hr/day)	24	U.S. EPA 2014 (OSWER Directive 9200.1-120)
ET _{res-a,h}	Exposure Time - resident adult hard surface (hr/day)	6	EPA 2003 (pg. D-4)
ET _{res-a,s}	Exposure Time - resident adult soft surface (hr/day)	10	EPA 2003 (pg. D-4)
ET _{res-c}	Air Exposure Time - resident child(hr/day)	24	U.S. EPA 2014 (OSWER Directive 9200.1-120)
ET _{res-c,h}	Exposure Time - resident child hard surface (hr/day)	6	EPA 2003 (pg. D-4)
ET _{res-c,s}	Exposure Time - resident child soft surface (hr/day)	10	EPA 2003 (pg. D-4)

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